



WP6 Innovation in Solid Organic Waste treatment and conditioning

CONSORTIUM MEETING, 25.05.2023

TH. MENNECART & WP6 PARTNERS

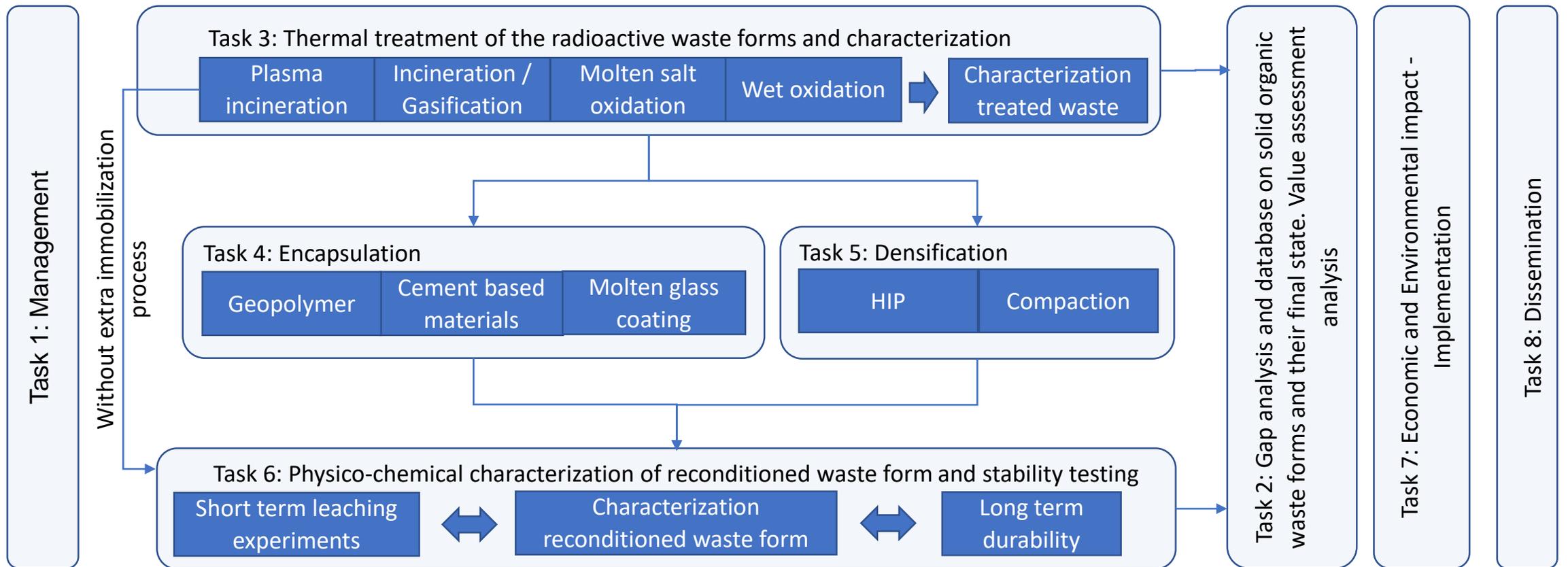


This project has received funding from the Euratom research and training programme 2019-2020 under grant agreement No 945098.

Overview of WP6 Objectives

- Perform a **gap analysis** during the first project year.
- **Demonstrate the reliability** of alkaline binders for conditioning of residues and secondary waste stemming from treatment of RSOW.
- **Verify the matrix performance** of conditioned final / ultimate waste according to a set of uniformed Waste Acceptance Criteria (WAC).
- **Improve understanding** of materials inventory before the thermal treatment and of the reconditioned waste once the conversion and immobilization has been achieved.
- Demonstrate **thermal treatment** methods leading to a significant **volume reduction** and to **safe reconditioned waste packages**.
- Deploy results for safe utilization by end users for **mathematical calculations** avoiding systematic experimental studies of the reconditioned waste.

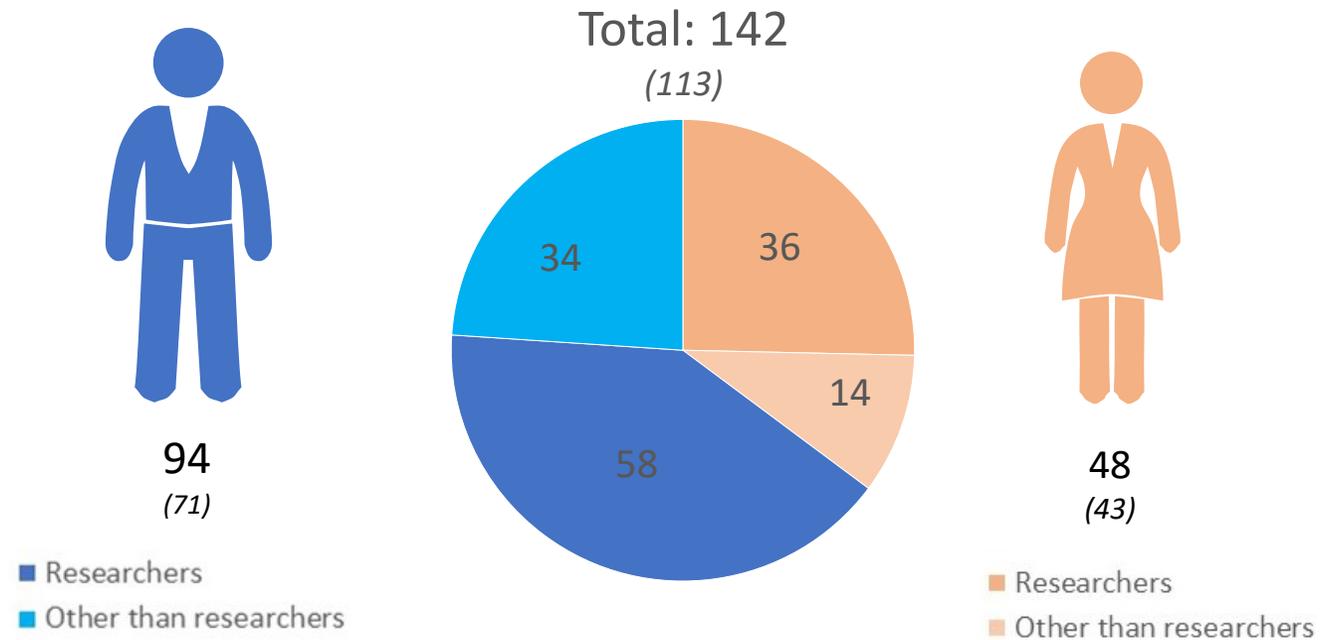
Work Package 6 Structure



Partners of WP6



Gender of researchers and other workforce



Planning

			2020				2021												2022												2023												2024							
			september	october	november	december	january	february	march	april	may	june	july	august	september	october	november	december	january	february	march	april	may	june	july	august	september	october	november	december	january	february	march	april	may	june	july	august	september	october	november	december	january	february	march	april	may	june	july	august
			1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48
			Year 1												Year 2												Year 3												Year 4											
WP 6	Innovations in solid organic waste treatment and conditioning	SCK•CEN	Active working months																																															
T6.1	WP Management	SCK•CEN	Active working months																																															
T6.2	Gap analysis and database on solid organic waste forms and their final state and value assessment	GSL	Active working months																																															
T6.3	Thermal treatment of the radioactive waste forms and characterisation of the treated / reconditioned waste	CEA	Active working months																																															
6.3.1	Thermal treatment of the Radioactive Solid Organic Wastes	CEA	Active working months																																															
6.3.2	Characterisation of the thermally treated / reconditioned wastes	CEA	Active working months																																															
T6.4	Immobilisation of the treat wastes by geopolymer or cement-based materials encapsulation or by molten glass coating	CVR	Active working months																																															
6.4.1	Geopolymer	CVR	Active working months																																															
6.4.2	Cement based materials	CSIC	Active working months																																															
6.4.3	Molten glass coating	CEA	Active working months																																															
T6.5	Densification	USFD	Active working months																																															
T6.6	Physico-chemical characterisation of reconditioned waste form and stability testing	VTT	Active working months																																															
6.6.1	Characterisation of reconditioned waste form	CIEMAT	Active working months																																															
6.6.2	Short term leaching experiments under different exposed conditions	CIEMAT	Active working months																																															
6.6.3	Long-term durability of reconditioned waste form	POLIMI	Active working months																																															
T6.7	Economic and Environmental impact - Implementation	GSL	Active working months																																															
T6.8	Dissemination	SCK•CEN	Active working months																																															
6.8.1	Reporting and guidelines		Active working months																																															
6.8.2	Dissemination activities to the scientific community and public		Active working months																																															



Deliverables (> [M36])

- ➔ [M36], D6.1. Summary report: Description of the thermal processes used for the thermal treatment of the RSOW and the physical properties and chemical composition of the resulting treated wastes.
- ➔ [M36], D6.2. Conditioning of ashes from the thermal treatment of RSOW by geopolymer or cement based materials encapsulation or by molten glass coating.
- [M42], D6.3. Economic, environmental and disposability impacts of novel treatment technologies for low-level and intermediate-level solid organic wastes.
- [M47], D6.4. Implemented database: Matching the chemical composition of the investigated reconditioned wastes with the initial waste stream.
- [M47], D6.5. Densification techniques test report. Description of the process, the type of treated waste immobilised and the physical properties and chemical composition of the resulting materials.
- [M47], D6.6. Final report on the Physico – chemical characterisation of reconditioned waste form and stability testing.
- [M47], D6.7. Modelling tool to predict the behaviour of the reconditioned wastes based on the results obtained within the project.
- ✓ [M48], D6.8. Submission in scientific journals of at least 5 papers about the RSOW stability and durability tests after reconditioning.
- [M47], D5.7. Summary report “Use of geopolymer matrices for the immobilisation of liquid and solid radioactive organic wastes”.

Milestones

- ✓ [M12], M6.1. Definition of the leaching procedure for the short term experiments and the long term durability experiments (6.2 & 6.3).
- ✓ [M22], M6.2. Delivery campaign of the goepolymer samples.
- ✓ [M22], M6.3. Delivery campaign of the cement based materials samples.
- ✓ [M24], M6.4. Delivery campaign of wet oxidation samples for HIP tests.
- ✓ [M24], M6.5. Intermediate report on densification (HIP, compaction).
- ✓ [M24], M6.6. LCA Case Study Input to WP2.
- ✓ [M29], M6.7. Intermediate report on characterisation of the durability related properties of conditioned waste form.
- ➔ [M34], M6.8. Report on feasibility demonstration of the Wet oxidation and the Molten salt oxidation routes for the treatment of RSOW: Description of the processes and basic physico-chemical properties of the reconditioned waste form.
- [M42], M6.9. Value assessment workshop.
- [M46], M6.10. Final report on short term leaching experiments.
- [M46], M6.11. Final report on long term durability of conditioned waste form.

Milestone 44 – LCA case study input to WP2

- Focus on treatment of ashes arising from the IRIS process:
 - Hot-Isostatic Pressing (University of Sheffield)
 - Encapsulation in Geopolymer (Politecnico di Milano)
 - Compaction (CEA)
- Delivered in M24



Milestone 44
LCA Case Study Input to WP2
Date 25.08.2022

Dissemination level: Public

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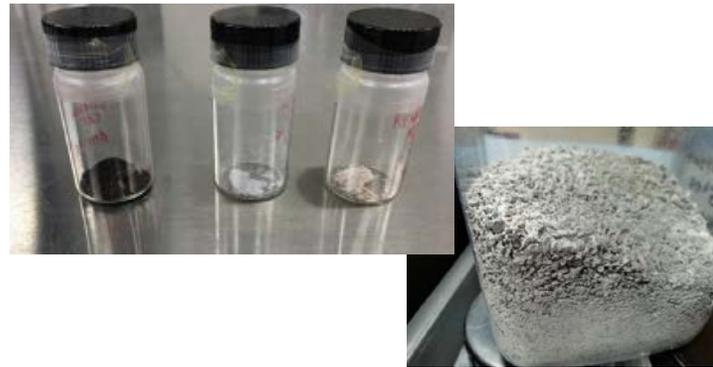
Task 6.2 Gap analysis and database on solid organic waste forms and their final state and value assessment analysis (GSL)

- Gap analysis achieved during the 1st year of the project
- Strategic overview of RSOW initiated during Theramin (confidential report)
- A questionnaire was circulated in mid-December 2020 (joint questionnaire covering data requirements for several PREDIS deliverables across work packages)
 - Not enough answer to build a database
 - High level summary of waste rather a database

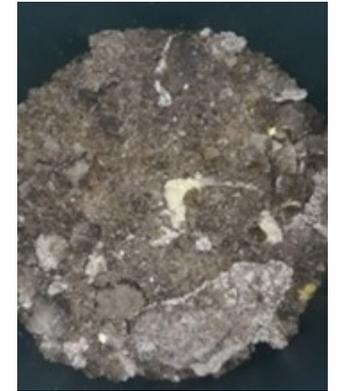
Task 6.3 Thermal treatment of the radioactive waste forms and characterization of the treated / reconditioned waste (CEA)

- Inventory of the available treated / reconditioned waste
- Sharing materials
 - IRIS ashes from CEA
 - MSO residues from CVRez
 - Thermally treated IERs from CIEMAT
- Characterization of the treated materials

Incineration / Gasification



Molten Salt Oxidation



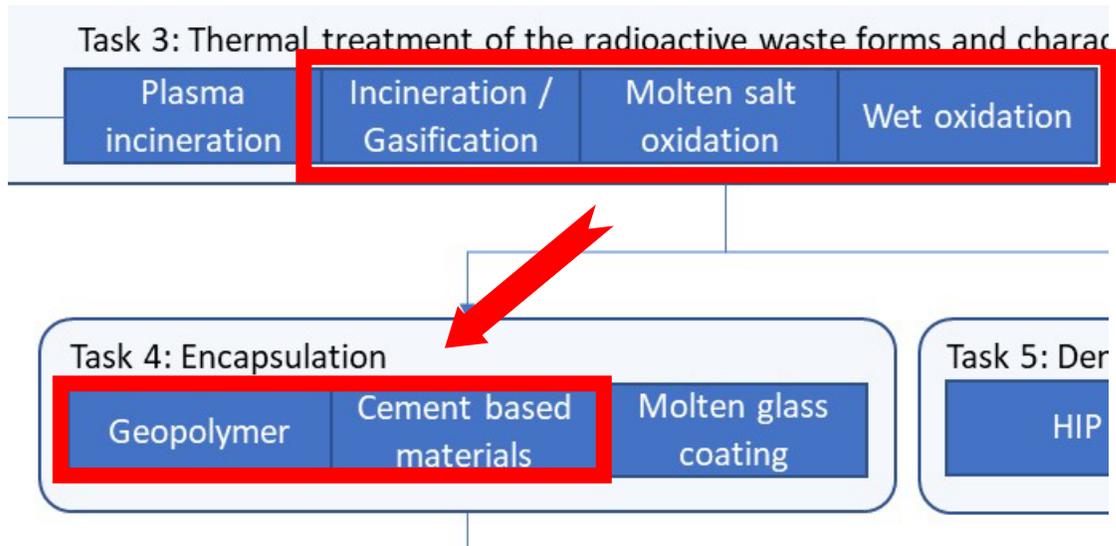
Plasma Incineration



Wet Oxidation

Task 6.4 Immobilization of the treated waste by geopolymer or cement based materials encapsulation or by molten glass coating (CVRez)

Different treated waste requires different immobilisation matrices design



Cement based materials vs Geopolymer
Optimization of the formulation
Use of alternative materials
About 20 –30 % of waste loading whatever the option



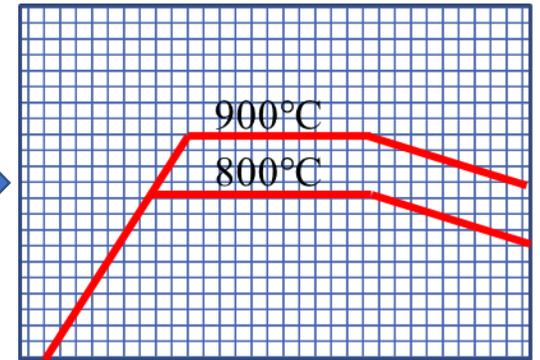
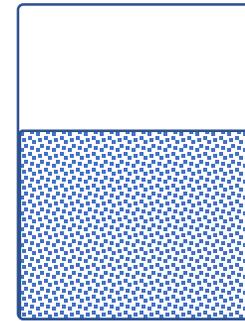
POLITECNICO MILANO 1863



Task 6.4 Immobilization of the treated waste by geopolymer or cement based materials encapsulation or by molten glass coating (CVRez)



+



Mix ratio ashes/glass
from 20/80 to 80/20

Weight load up to **40%w**



- ✓ Soft process
- ✓ Avoid radionuclide volatilization
- ✓ Coating of the waste
- ✓ No pressing, under air condition
- ✓ Obtaining a monolithic form

Task 6.5 Densification (USFD)

- Compaction tests: pelletizing and granulation



- Hot Isostatic Pressing
 - IRIS ashes (4x formulations)
 - Polymer WIP
 - Wet oxidation sludge (in progress)
 - KIPT HIP at USFD



Task 6.6 Physico-chemical characterization of reconditioned waste forms and stability testing (VTT)

- **Leaching experiments:**
 - Common protocol, national recommendation
 - After gamma irradiation, doped materials

About 400 samples

- **Harmonization of the results (proposition).** Excel file to calculate:
 - Normalized Loss
 - Cumulative Fraction Leached
 - Effective Diffusion Coefficient
 - Leachability Index

Task 6.7 Economic and Environment impact - Implementation

Deliverables: Economic, environmental and disposability impacts of novel treatment technologies for low-level and intermediate-level solid organic waste [M42](GSL)

- Disposability Assessment Approach

Dissemination and Reporting

Conferences & Workshops

- **NUWCEM** 2022, 4th International Symposium on Cement-Based Materials for Nuclear Wastes
- **FISA 2022 & EURADWASTE '22**, 10th Euratom Conference on Reactor Safety & 10th Euratom Conference Radioactive Waste Management
- **CNMAT** 2022, XVI Congreso Nacional de Materiales,
- **NENE** 2022, 1st International Conference Nuclear Energy for New Europe
- **IGD-TP** Symposium and Webinar: The role of optimization in radioactive waste geological disposal programmes
- **NUMAT** 2022, The Nuclear Materials Conference
- **Materials Research Symposium** Fall Meeting 2022
- **Migration** 2023
- **6th International Workshop on Mechanisms and Modelling of Waste / Cement Interactions & EURAD** –WP CORI Final Workshop
- The 16th International **Congress on the Chemistry of Cement** 2023

13 presentations

Dissemination and Reporting

Scientific papers

D6.8 (M48). Submission in scientific journals of at least **5 papers** about the RSOW stability and durability tests after reconditioning

1. “Fenton-like treatment for reduction of simulated carbon-14 spent resin”, M.A. Hafeez, J. Jeon, S. Hong, N. Hyatt, J. Heo, W. Um
Journal of Environmental Chemical Engineering (2021), 9-1, <https://doi.org/10.1016/j.jece.2020.104740>
2. “Stability of SrCO₃ within composite Portland-slag cement blends”, S.A. Walling, L.J. Gardner, D.P. Prentice, M.C. Dixon Wilkins, A.A. Hammad, W. Um, N.C. Hyatt
Cement and Concrete Composites, 135, 104823 (2023), <https://doi.org/10.1016/j.cemconcomp.2022.104823>
3. “Design of sustainable geopolymetric matrices for encapsulation of treated radioactive solid organic waste”, A. Santi, E. Mossini, G. Magugliani, F. Galluccio, E. Macerata, P. Lotti, G. D. Gatta, D. Vadivel, D. Dondi, D. Cori, H. Nonnet, M. Mariani
Frontiers in Materials (2022), <https://doi.org/10.3389/fmats.2022.1005864>
4. “Solidification of molten salt from Molten Salt Oxidation technology”, S V. Galek, A. Černá, P. Pražák, T. Černoušek, M. Vacek, V. Berger, J. Hadrava
In preparation
5. “Effect of the incorporation of a molten salt waste from nuclear power plants in the development of geopolymers and Portland cement systems”, P. Perez-Cortes, I. Garcia-Lodeiro, F. Puertas, M. Cruz Alonso
In preparation in “Cement and Concrete Composites”
6. “Explorative scale-up of Fenton Oxidation and Geopolymer Encapsulation for the management of spent mixed bed ion exchange resins”, F. Galluccio, E. Mossini, A. Santi, G. Magugliani, M. Giola, E. Macerata, G. D. Gatta, P. Lotti, D. Cori, G. Bilancia, P. Peerani, and M. Mariani
In preparation in “Nuclear Engineering and Technology Journal”

Level technology – Ambition

Task	Technology (2019)	Ambition after 4 years	Current situation
3.1	Plasma incineration. Treatment of IER, cement concentrate, others	Increase the SHIVA technology level. TRL from 5 to 6	✘
3.1	Incineration / Gasification. Treatment of IER	Feasibility demonstration of the technology by incineration of (inactive) bituminized waste. TRL from 2 to 4	✘
3.1	Molten Salt Oxidation used for the treatment of Radioactive Liquid Organic Waste	Transposition of the technology to the treatment of RSOW (IER). Trials with (inactive?) IER and conditioning of the salt using goepolymer or cement based materials. TRL from 4 to 6	On-going
3.1	Wet Oxidation Route used for the destruction of dissolved organic contaminants	Development and optimization of the process for the destruction of IER leading to the complete recovery of the ¹⁴ C and associated radionuclide inventory into iron sludge. The sludge will be thermal treated for a complete immobilisation (task 5). TRL from 2 to 3 - 4.	On-going
4.1	Goepolymer immobilisation	Determination of the best geopolymer formulation for a safe and long term immobilisation of treated wastes after thermal treatment (e.g. ashes, salt). TRL from 2 - 3 to 5	On-going
4.2	Cement based materials immobilisation	Determination of the best cement based materials formulation for a safe and long term immobilisation of treated wastes after thermal treatment (e.g. ashes, salt). TRL from 3 to 5	On-going
4.3	New technique: Molten glass coating	Feasibility demonstration of the glass coating for the immobilisation of ashes after incineration of IER at the lab scale. TRL from 1 to 4.	On-going
5	HIP technology	Increase the technology level using radiotracers or radioactive samples. TRL from 2 to 4	On-going
5	Compaction assisted by thermal treatment	Feasibility demonstration of densification of ashes coming from incineration process by compaction, eventually with adjuvants and temperature. TRL from 1 to 3 - 4	On-going

Scientific presentations – Flash talk

- “RSOW treatment using Molten Salt Oxidation process”
Vojtech Galek (CVRez)

- “Immobilization of molten salt residue using alkali-activated and cement-based materials”
 - Lander Frederickx, Eduardo Ferreira, Quoc tri phung (SCK CEN)

- Flash talk
 - Gianni Vettese (UH)



RSOW treatment using Molten Salt Oxidation process

VOJTECH GALEK

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This project has received funding from the Euratom research and training programme 2019-2020 under grant agreement No 945098.

Laboratory of Hazardous and Radioactive Waste Processing

- Molten Salt Oxidation (MSO) pilot plant for processing organic waste
- Evaporation/drying (ETL) of liquid waste (to the powder form)
- Equipment for solidification into geopolymer and polysiloxane matrices (GPS)



MSO Technology

- Volume reduction
- Flameless oxidation
- Alkaline-based carbonates
- Scrubber for acidic gases
- Stable heat transfer
- Catalyst-like environment



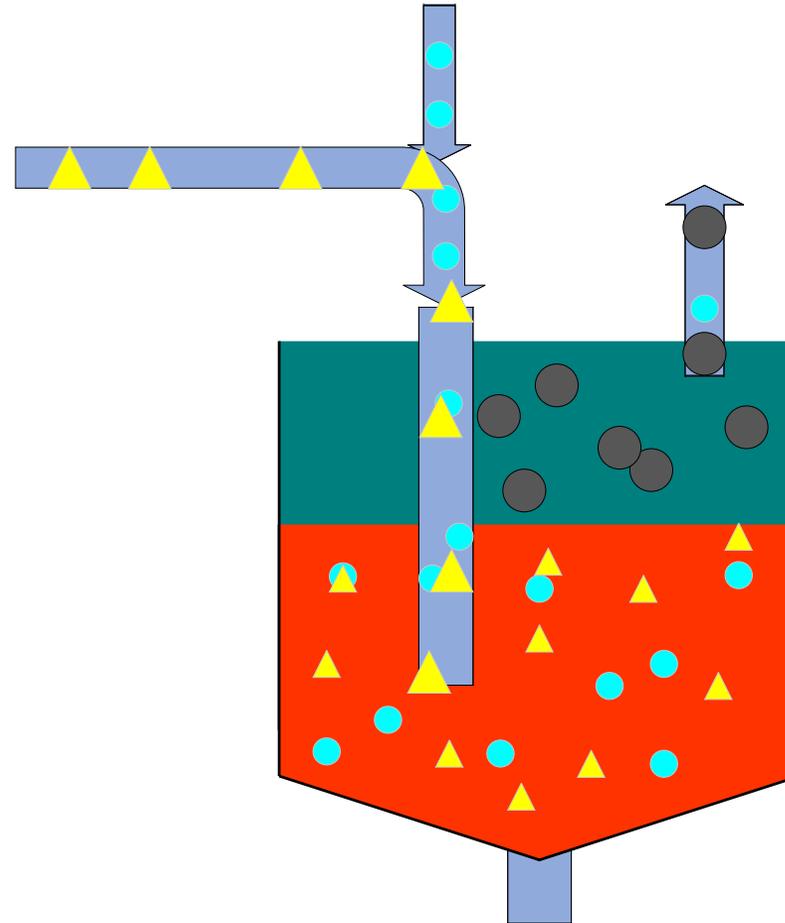
MSO Technology – Processed waste

- SORW: ion resins
- LORW: lubricants, organic solvents, scintillation liquids and decontamination liquids
- Hazardous waste: PCE, PCB, PVC, HCB, CFC-12, CN, PCDD/F
- Waste needs to be fully analysed



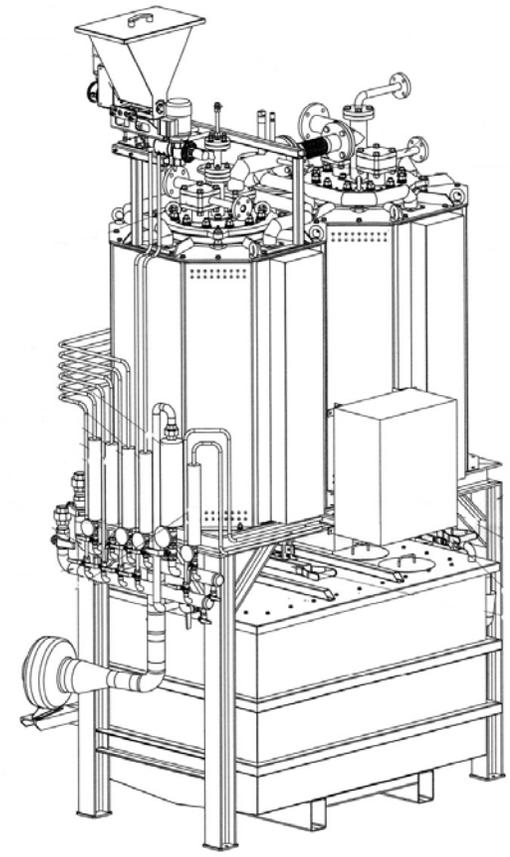
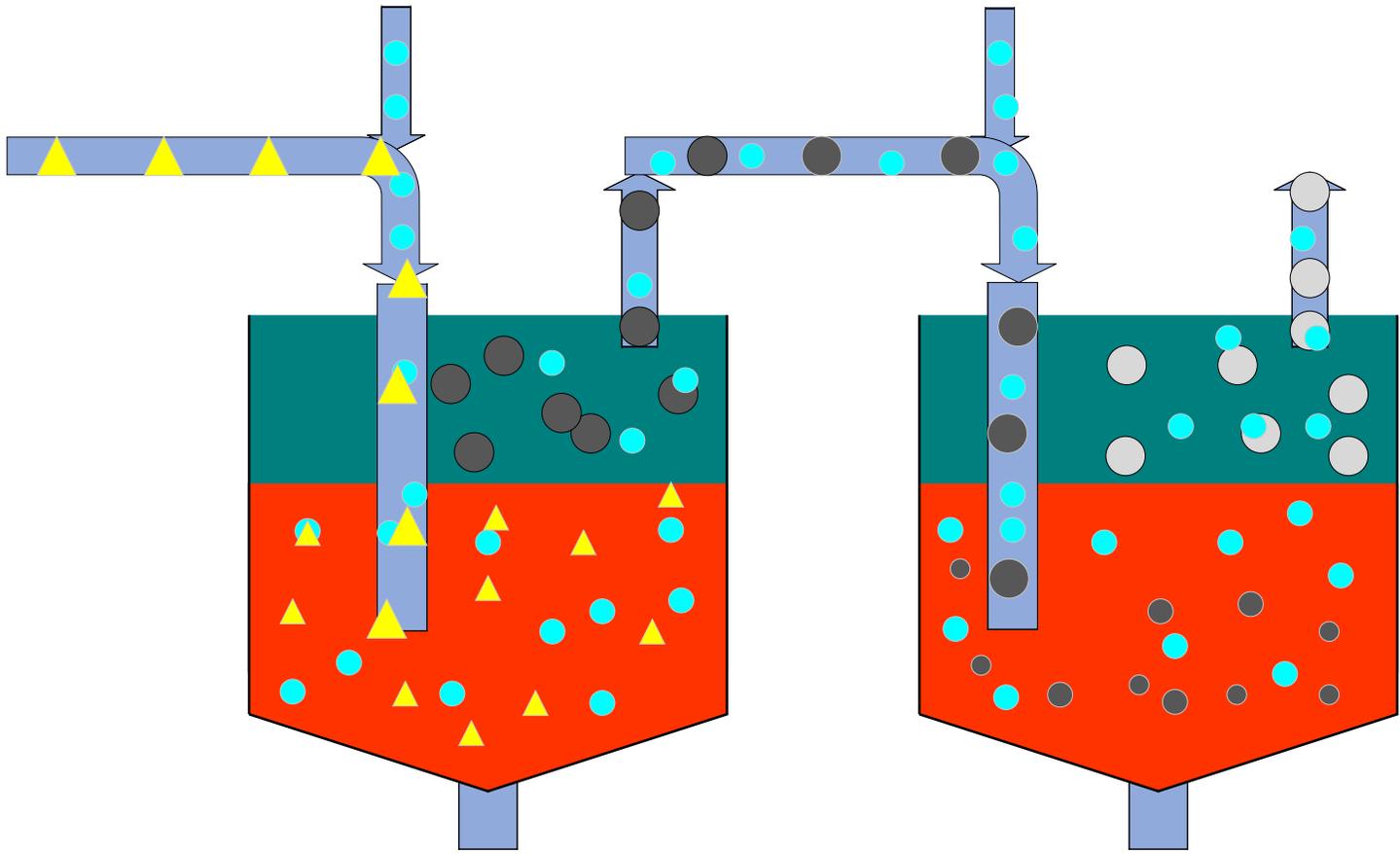
Principle of MSO

- Low dosing rate
- Low oxidation efficiency
- Increased time for total oxidation
- Low applicability

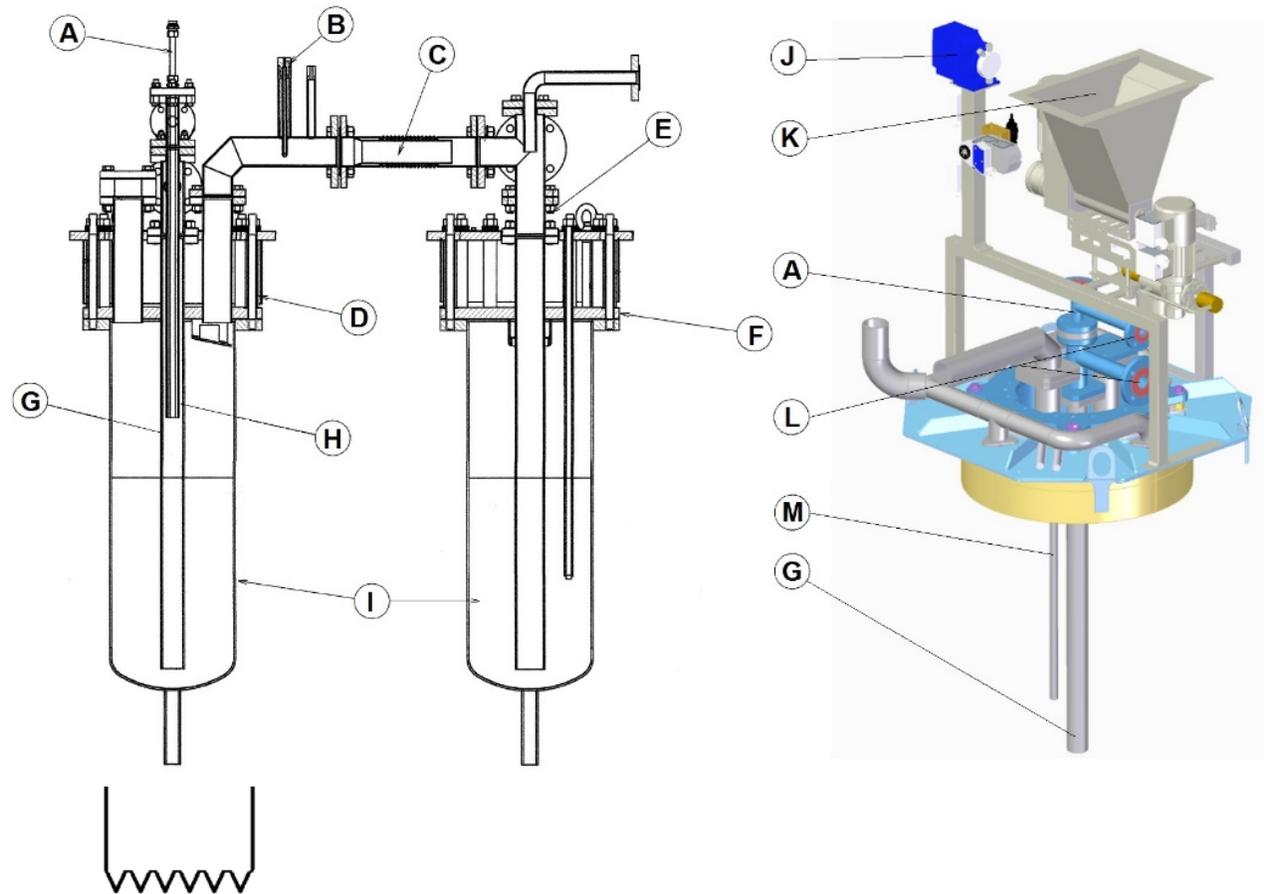




Principle of MSO



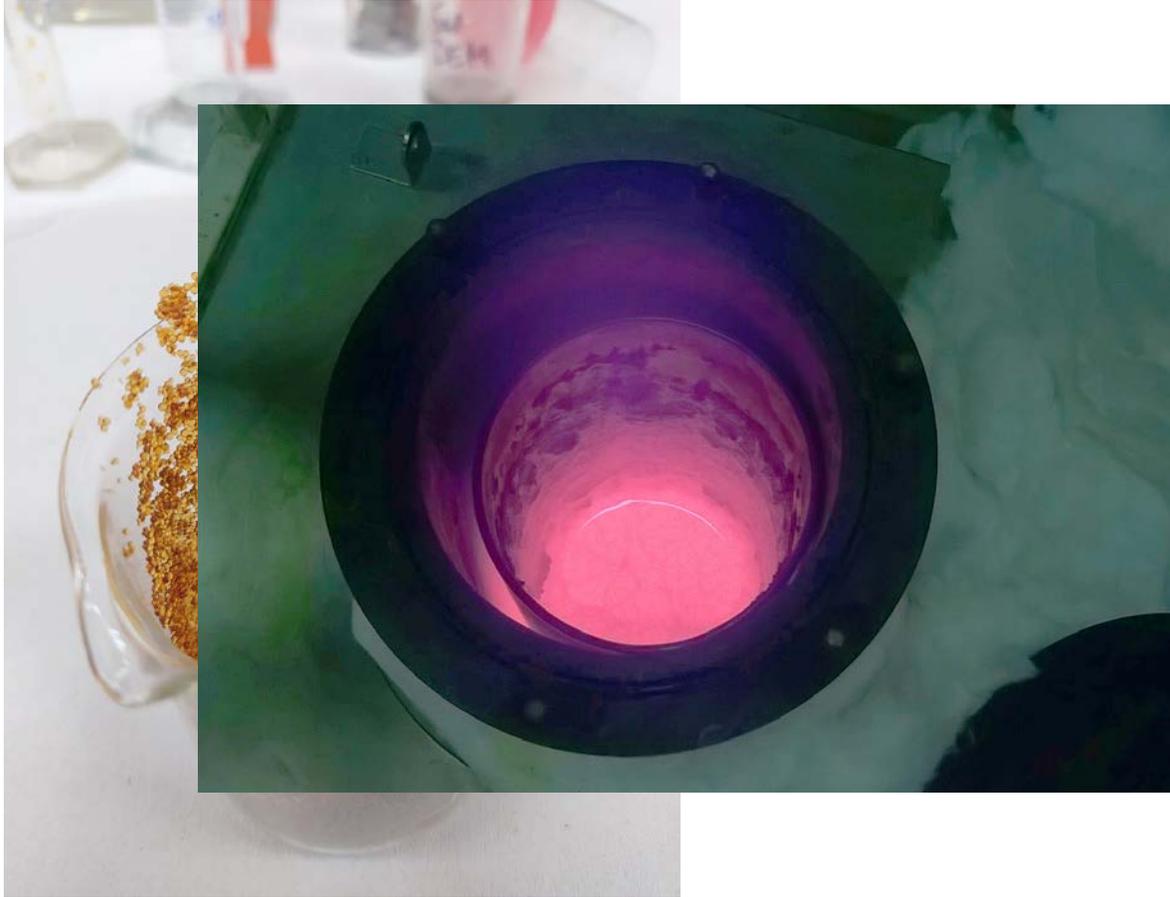
Principle of MSO



What is the MSO waste



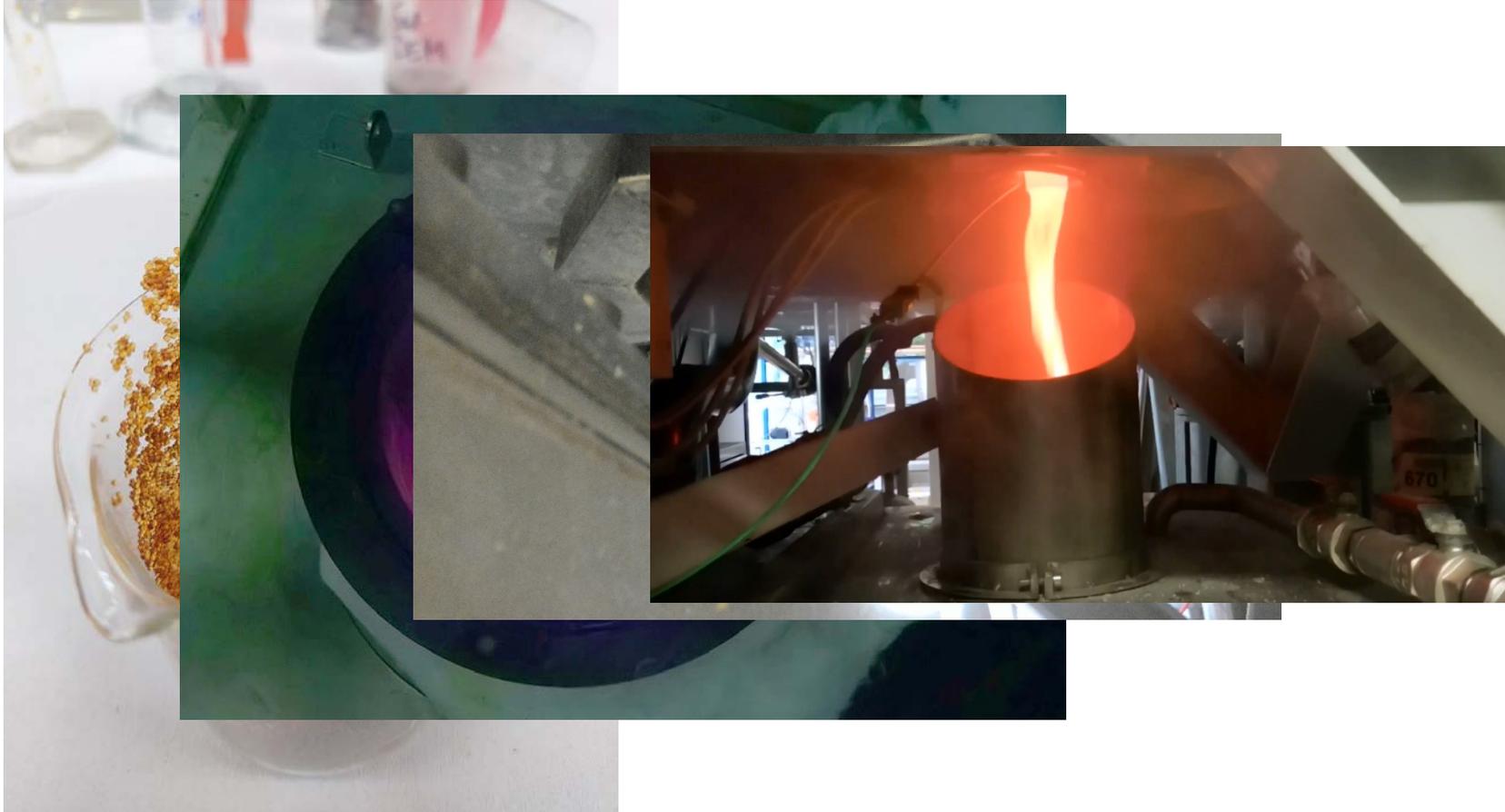
What is the MSO waste



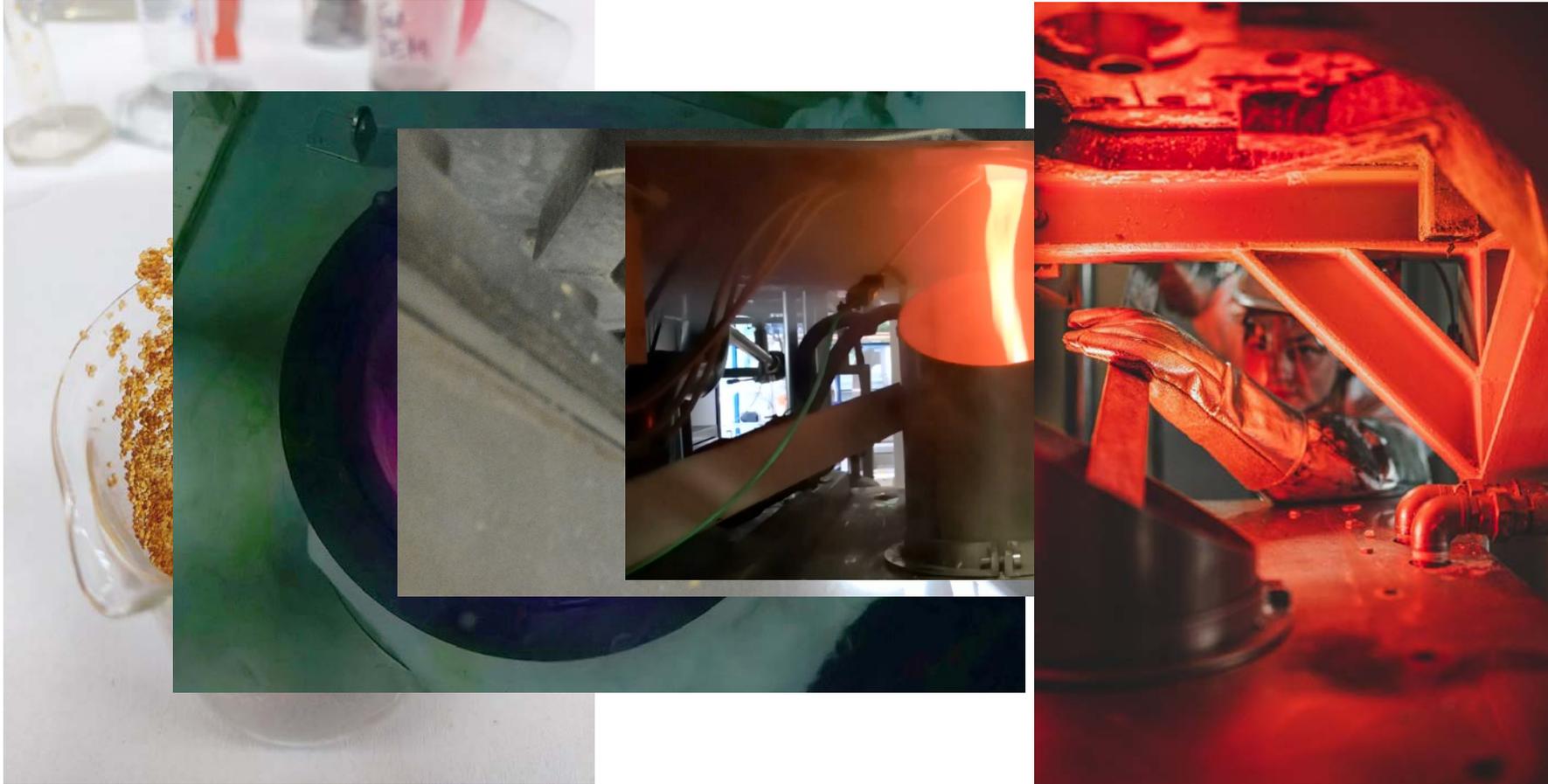
What is the MSO waste



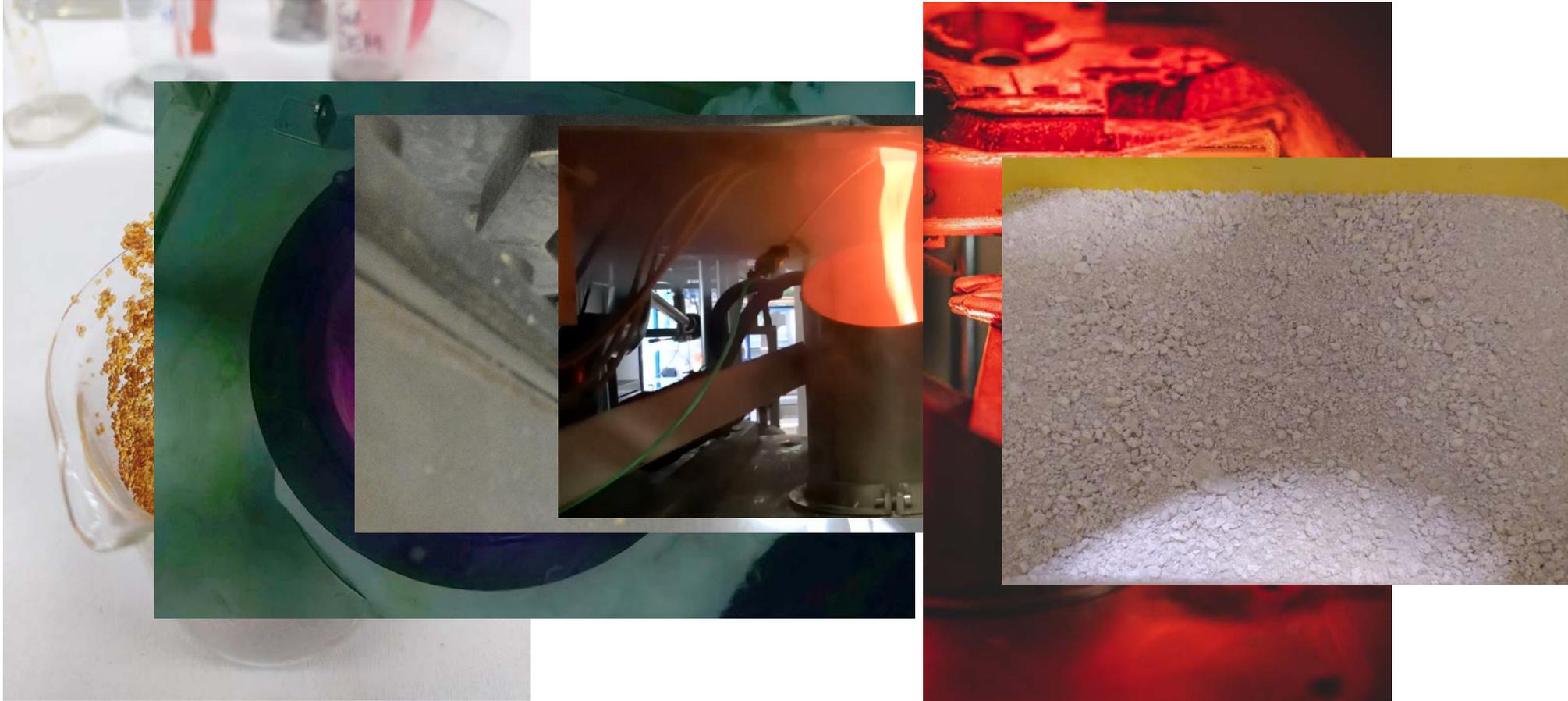
What is the MSO waste



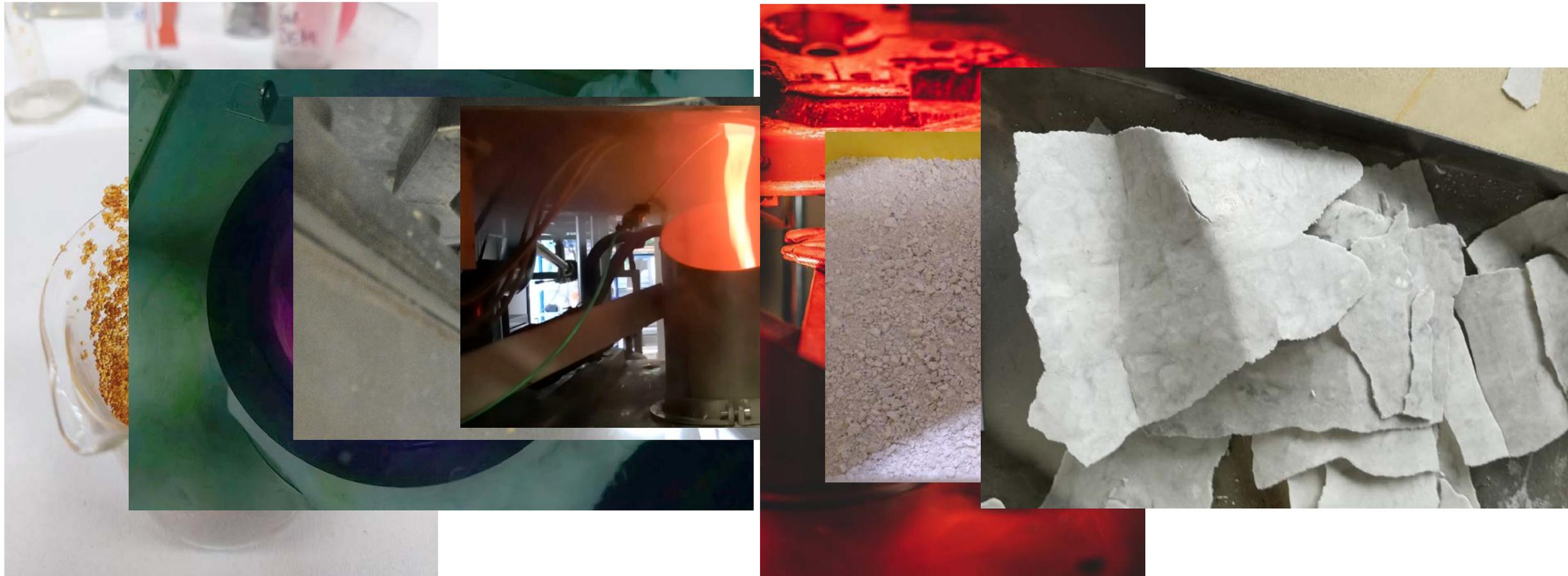
What is the MSO waste



What is the MSO waste



What is the MSO waste







Immobilization of molten salt residue using alkali-activated and cement-based materials

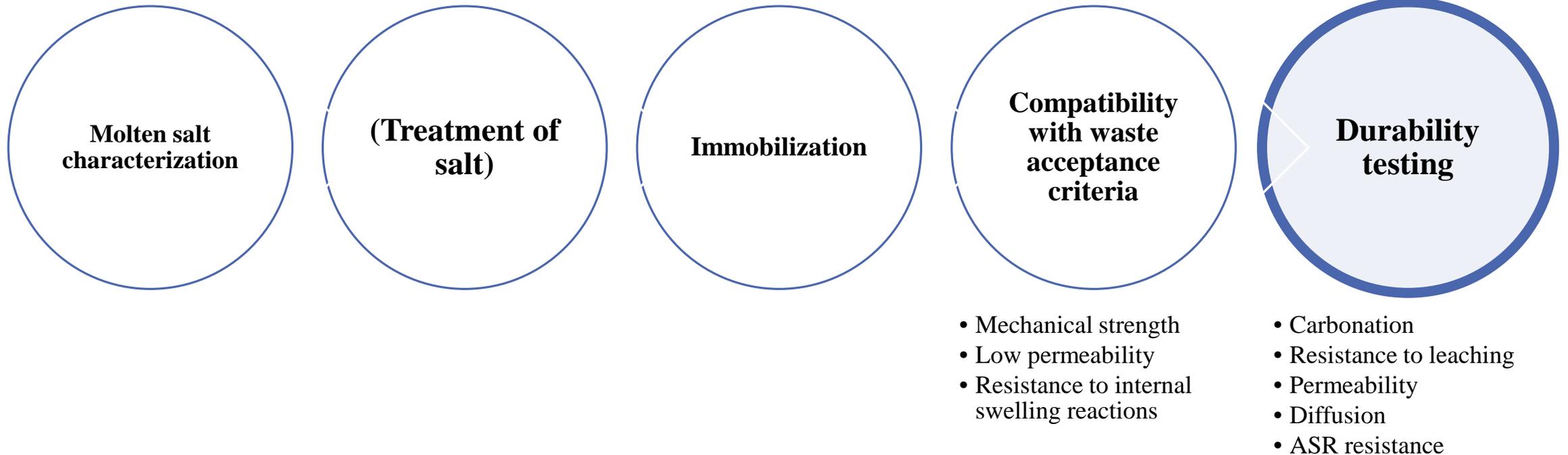
LANDER FREDERICKX, EDUARDO FERREIRA, QUOC TRI PHUNG

sck cen



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Immobilization of molten salt: roadmap



Matrices under consideration

Blended cementitious system



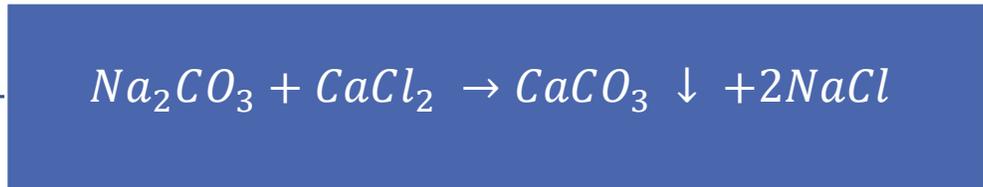
- CEM I / BFS / Silica fume / lime / limestone
- Varied waste loading and w/c ratio
- Proven matrix for other types of radwaste

Alkali-activated materials

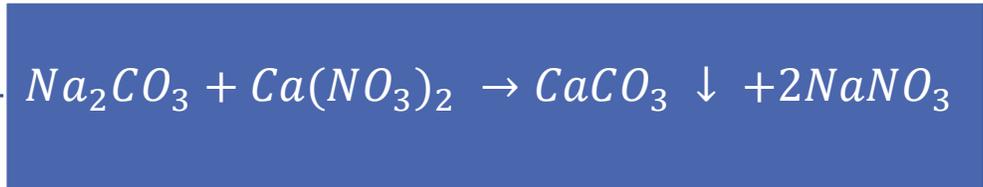


- Metakaolin / BFS precursor
- Resistant to DEF/ASR
- Encapsulation of waste in 3D (C)NASH structure

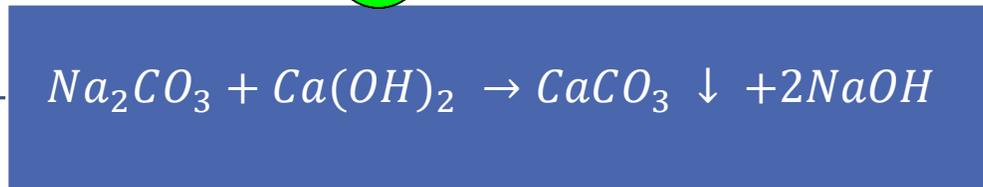
Pre-treatment is necessary!



$s = 75 \text{ g/L}$



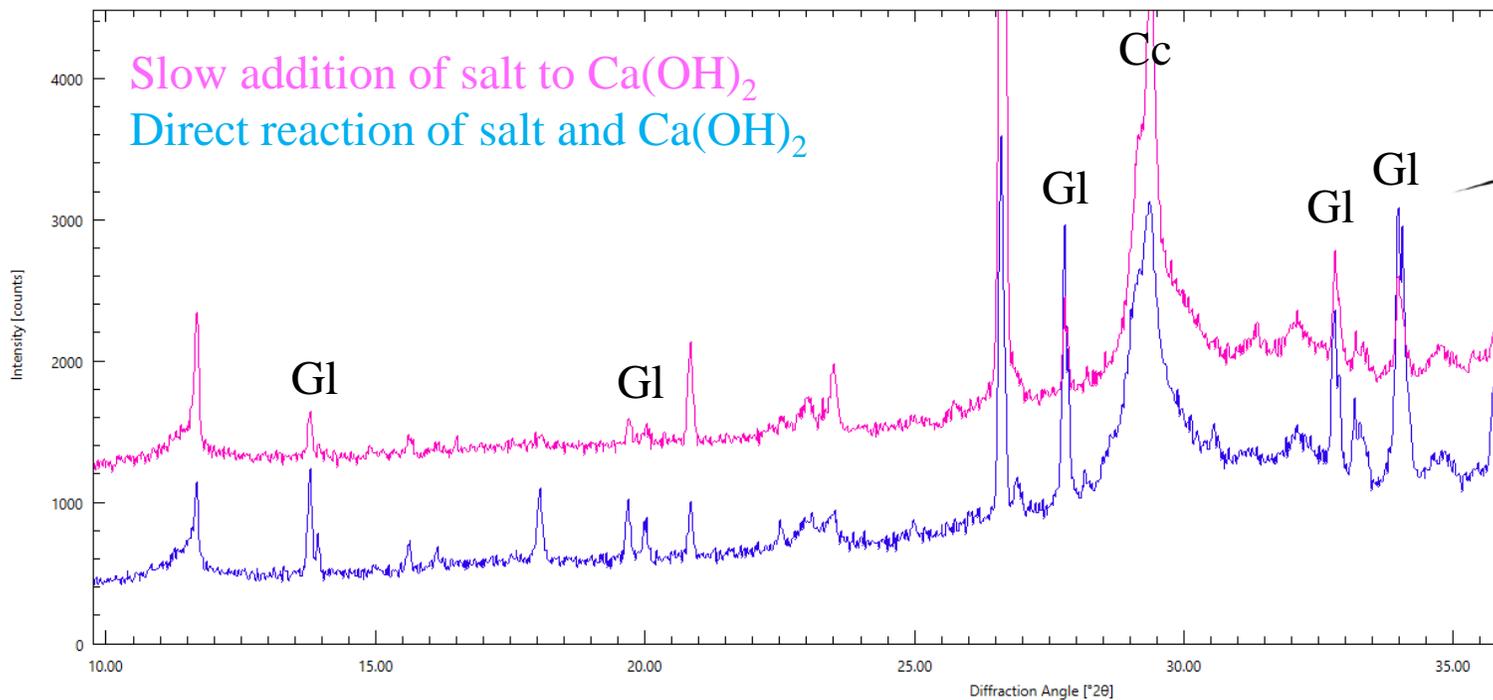
$s = 1212 \text{ g/L}$



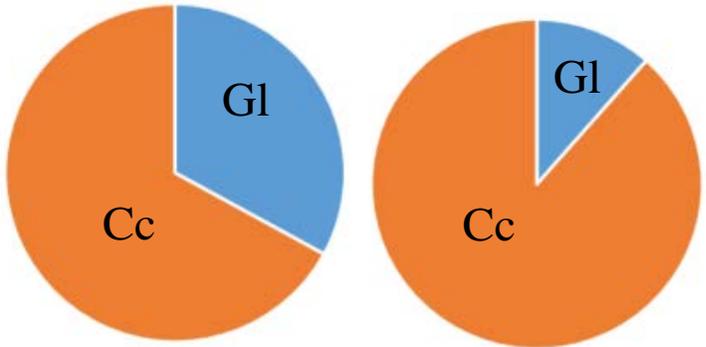
$s = 1.73 \text{ g/L}$



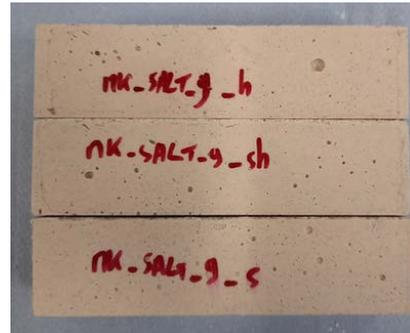
Transformation of salt by Ca(OH)_2



Gaylussite
 $\text{Na}_2\text{Ca}(\text{CO}_3)_2 \cdot 5\text{H}_2\text{O}$



Immobilization of $\text{Ca}(\text{OH})_2$ treated waste



Metakaolin geopolymer prisms with 10 wt.% salt, converted by $\text{Ca}(\text{OH})_2$



Alkali-activated BFS prisms with 10 wt.% salt, converted by $\text{Ca}(\text{OH})_2$



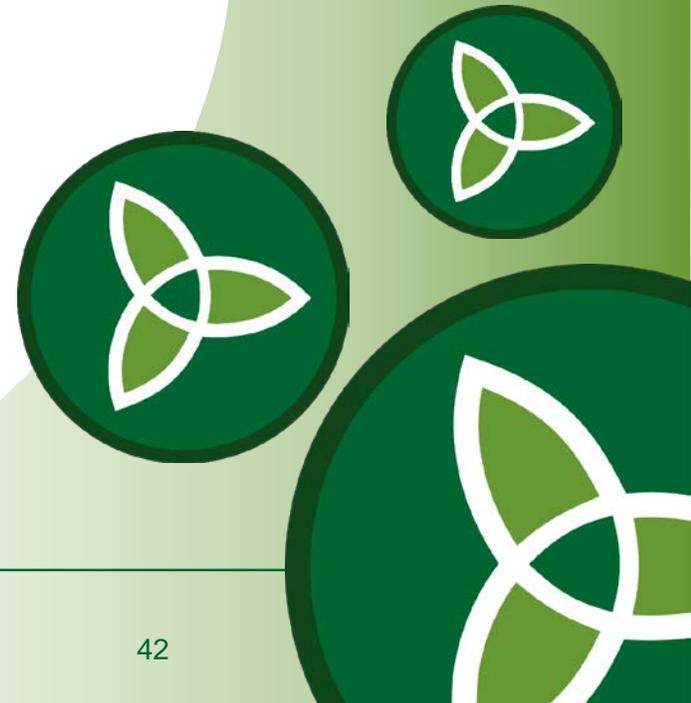
Blended system cubes with 10 and 14 wt.% salt, converted by $\text{Ca}(\text{OH})_2$

	Metakaolin		BFS		Blended system	
Waste loading (%)	10	15	10	20	10	14
Compressive strength (MPa)	25±1	13±2	42	37	15	15
Final setting time (hours)	>5 h	/	2h30m	1h15m	5h00m	1h17m
Notes	Efflorescence		/		/	

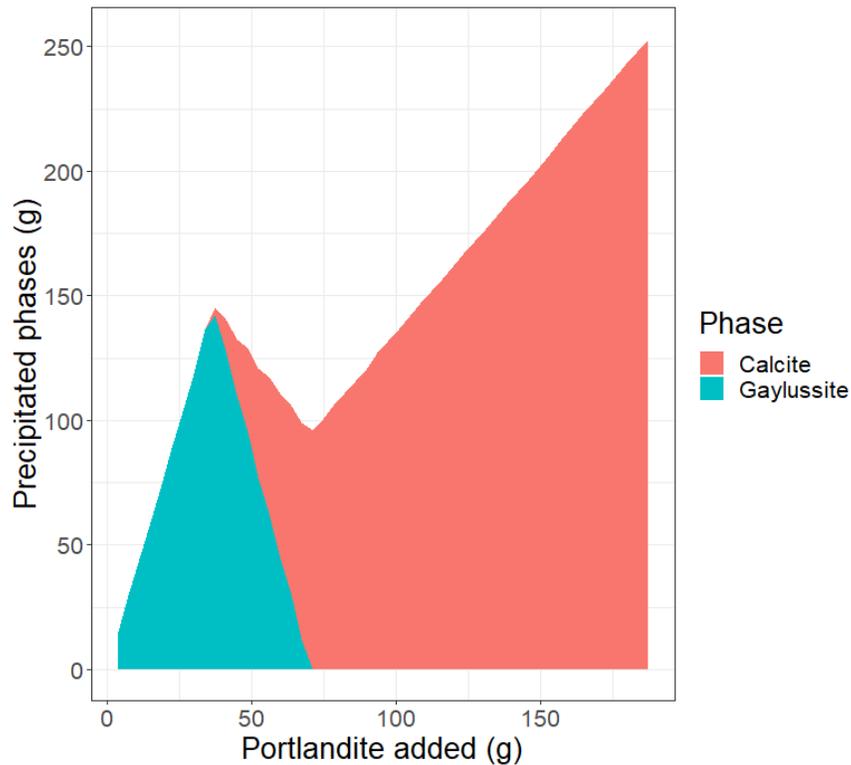


PREDIS

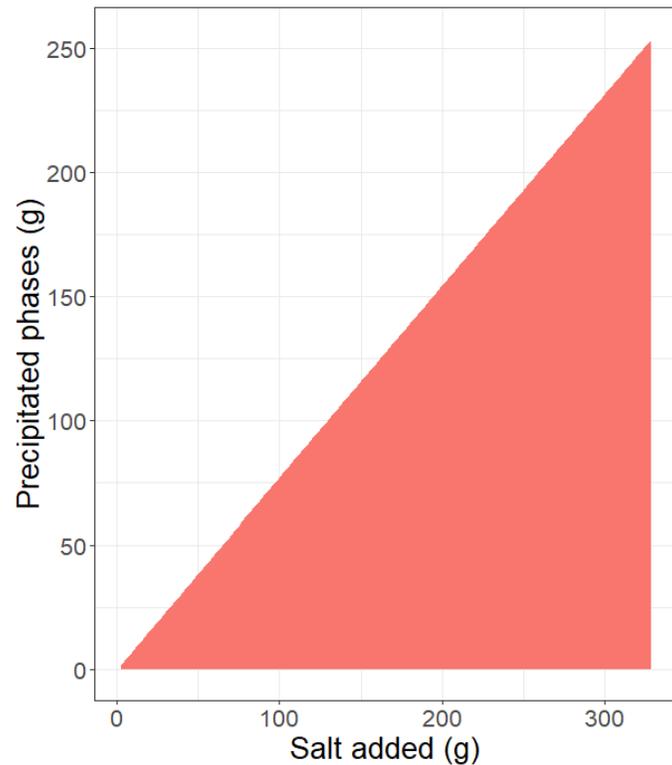
Thermodynamic modeling



Speciation of carbonate phases after pretreatment



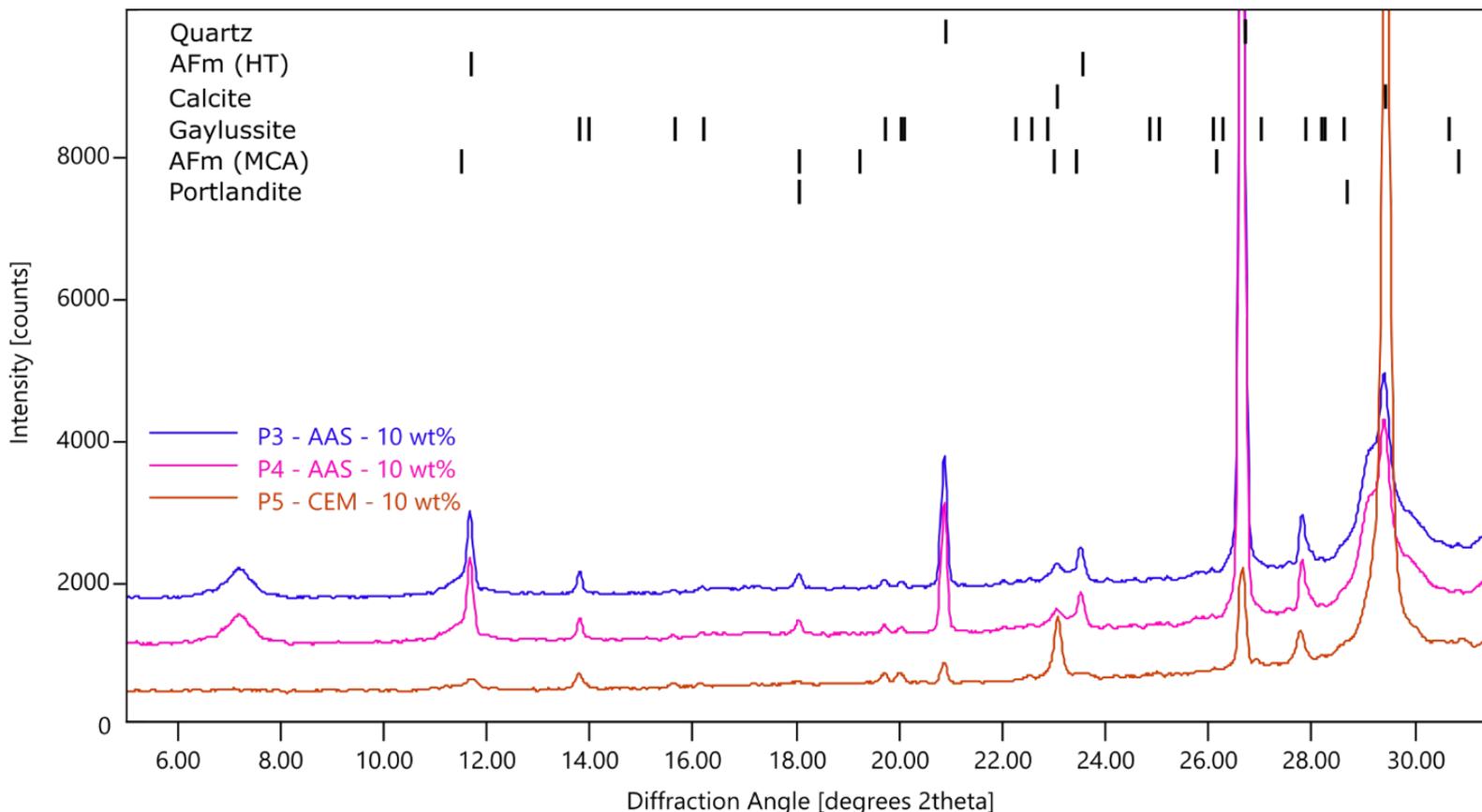
Stepwise addition of Ca(OH)₂ to salt



Stepwise addition of salt to Ca(OH)₂

- Unexpected presence of gaylussite after pretreatment
- Attributable to Na-rich environment in which Ca(OH)₂ is introduced
- Stepwise addition of Ca(OH)₂ promotes calcite formation

Carbonate phases in matrix



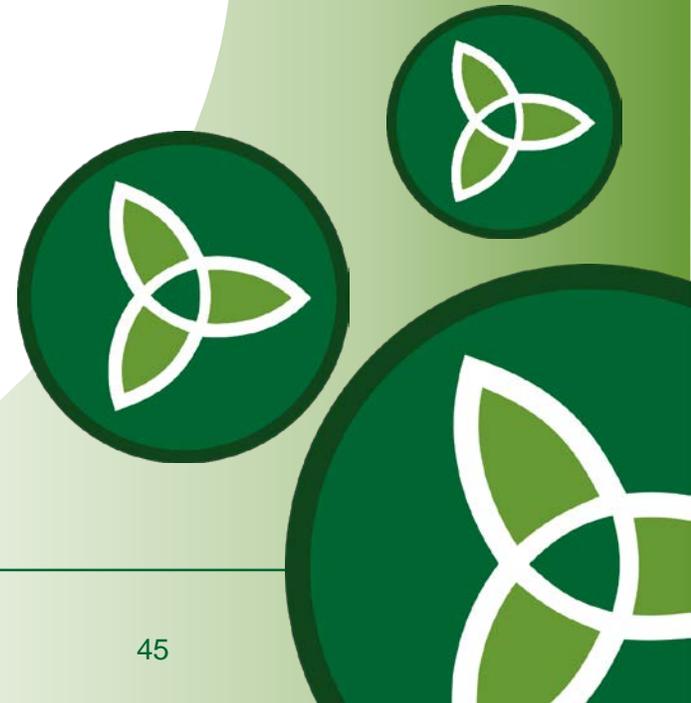
	P4	P5
Portlandite	0.0	0.0
AFm	2.2	1.7
C(A)SH	51.1	47.1
Unreacted Slag	17.3	10.4
Gaylussite	12.2	0.0
Calcite (aggregate)	0	21.6
Calcite (salt)	3.3	16.4
Quartz	13.9	2.9

- Gaylussite still occurs even after stepwise addition of $\text{Ca}(\text{OH})_2$
- AAM (P4): gaylussite is thermodynamically stable
- Cementitious materials (P5): gaylussite is not stable



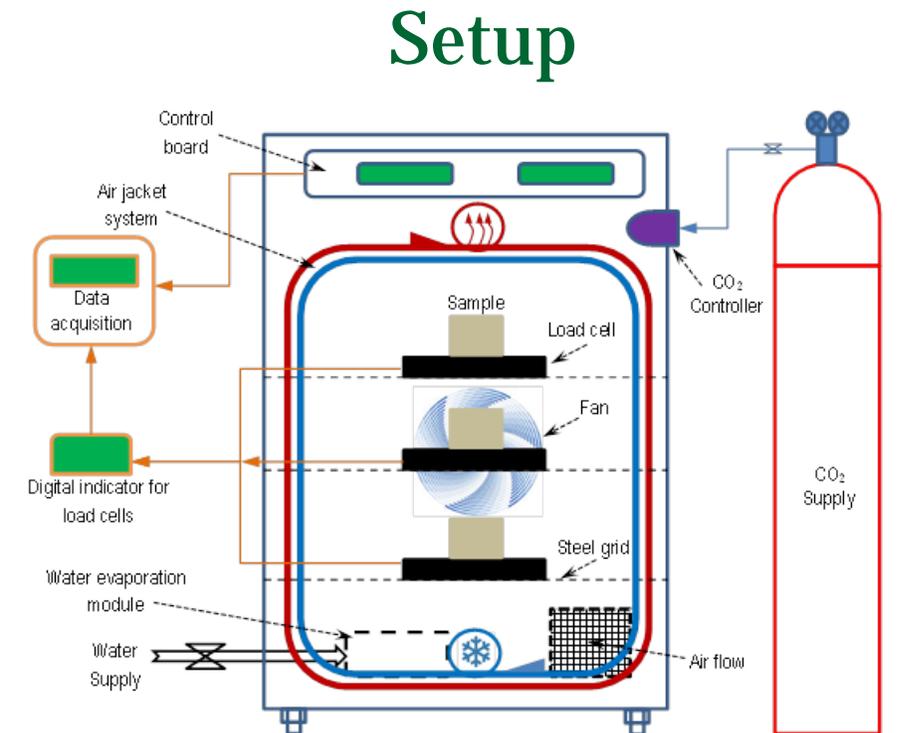
PREDIS

Durability



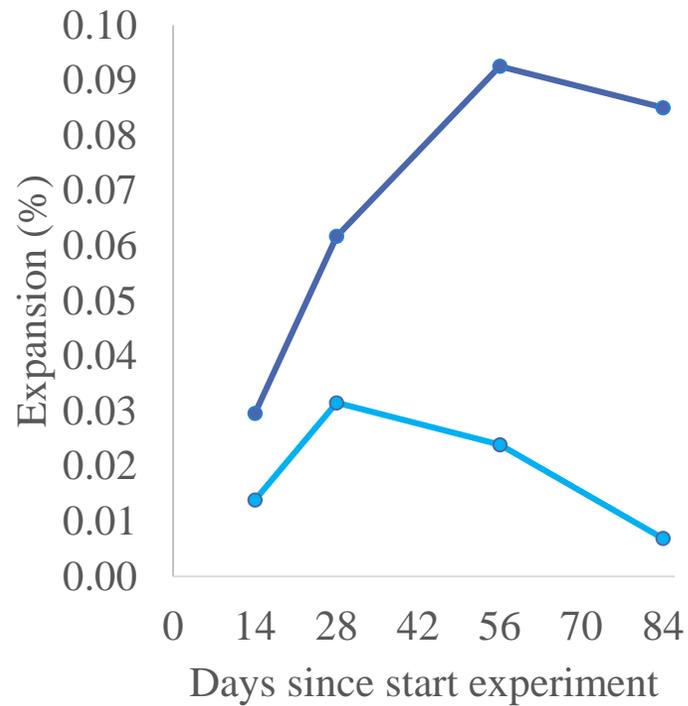
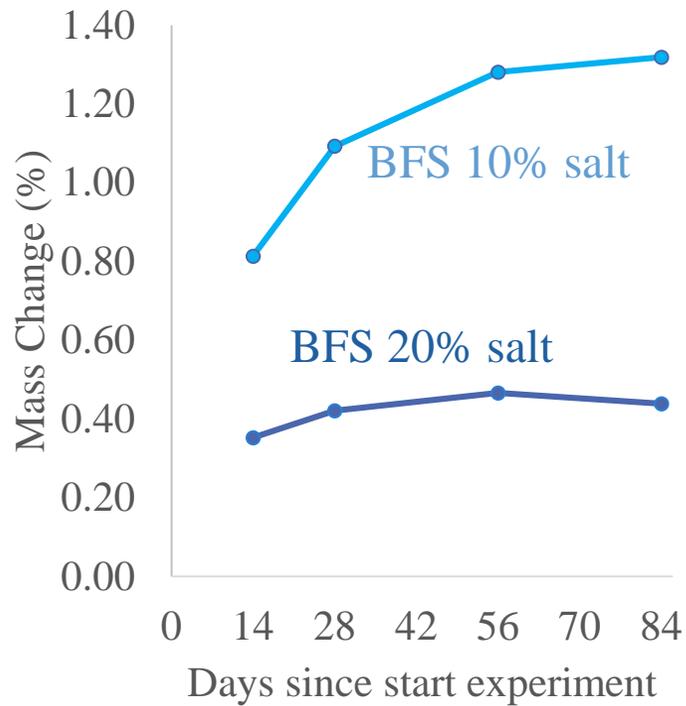
Carbonation experiments

- Samples cured for 28 days
- Preconditioning at 60% RH, 20°C until achievement of stable mass
- Exposure to 1% CO₂, 60% RH, 20°C
- 3 Months
- Intermediate follow up
 - Measurements: strength, length
 - Characterization



Carbonation experiments

Mass & Length

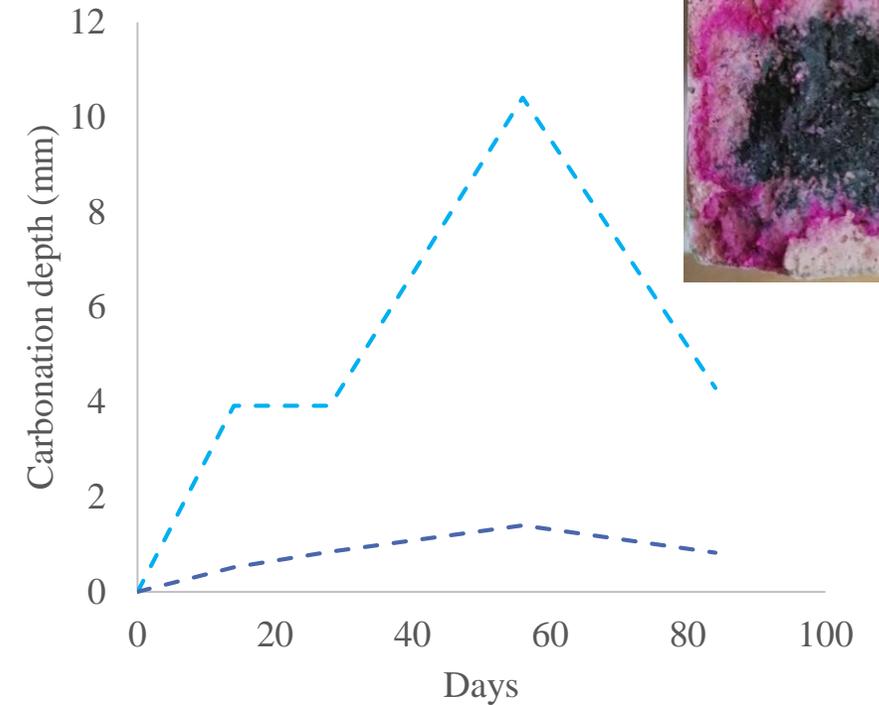
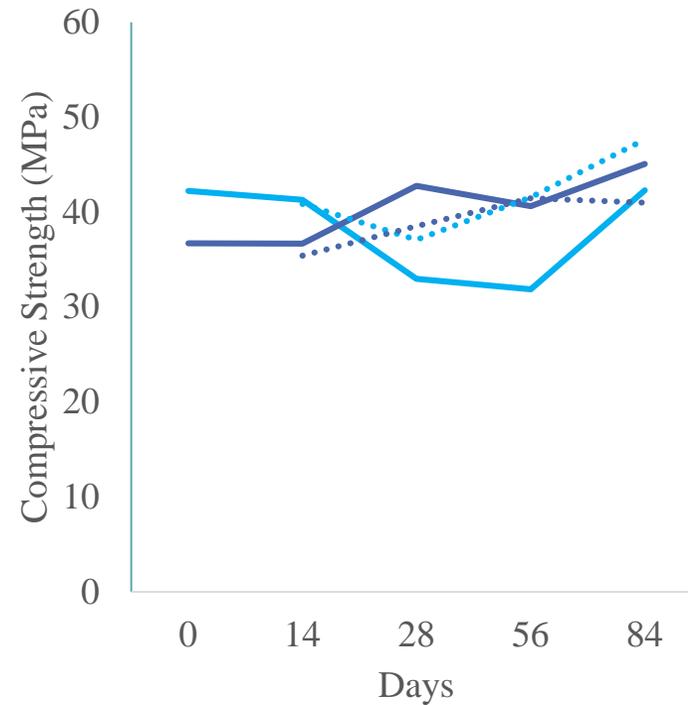
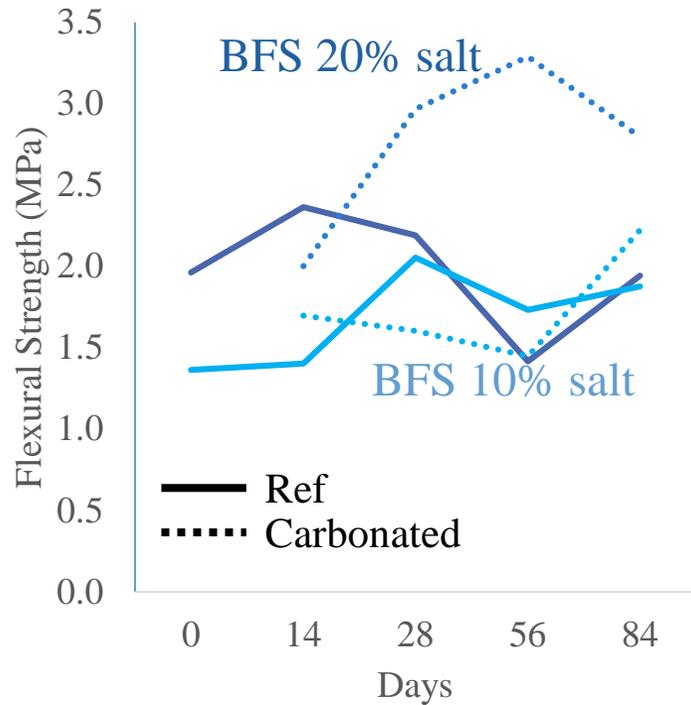


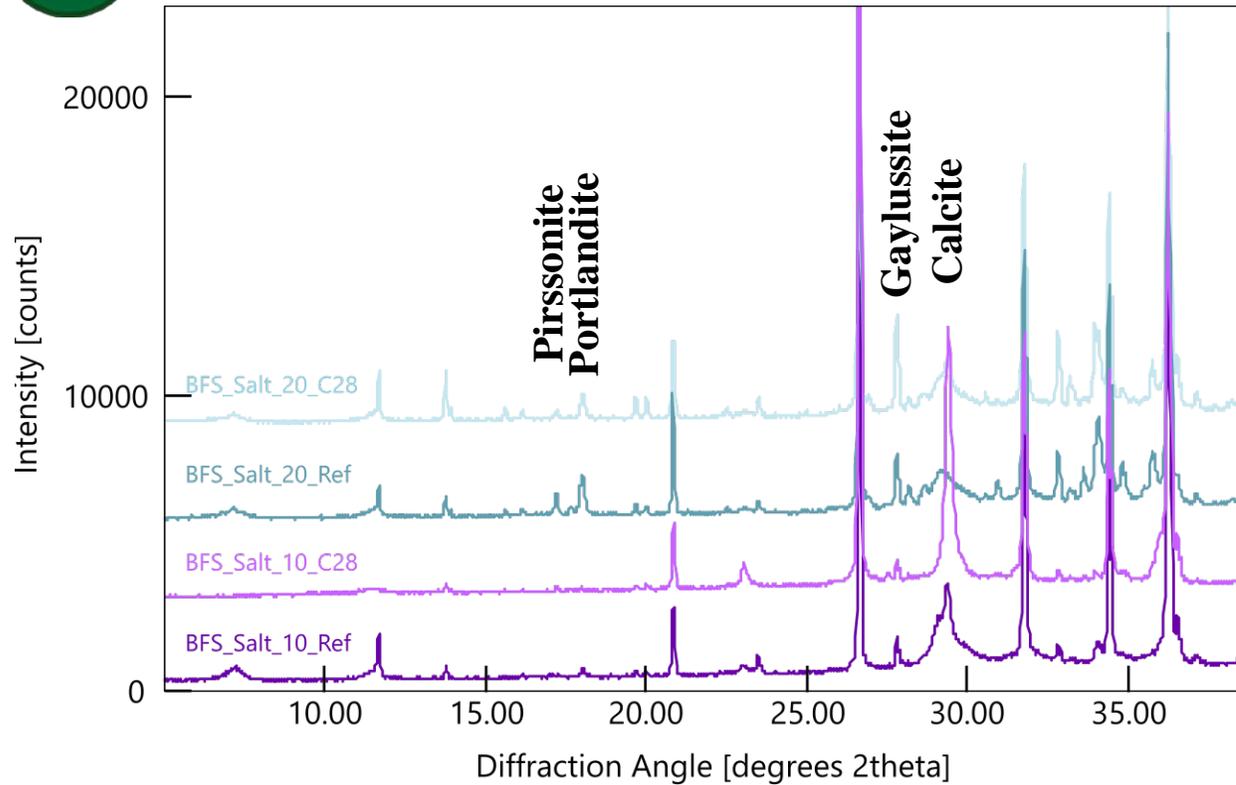
Mass increase
 \approx
 Ingress of CO₂
 \neq
 Expansion of samples



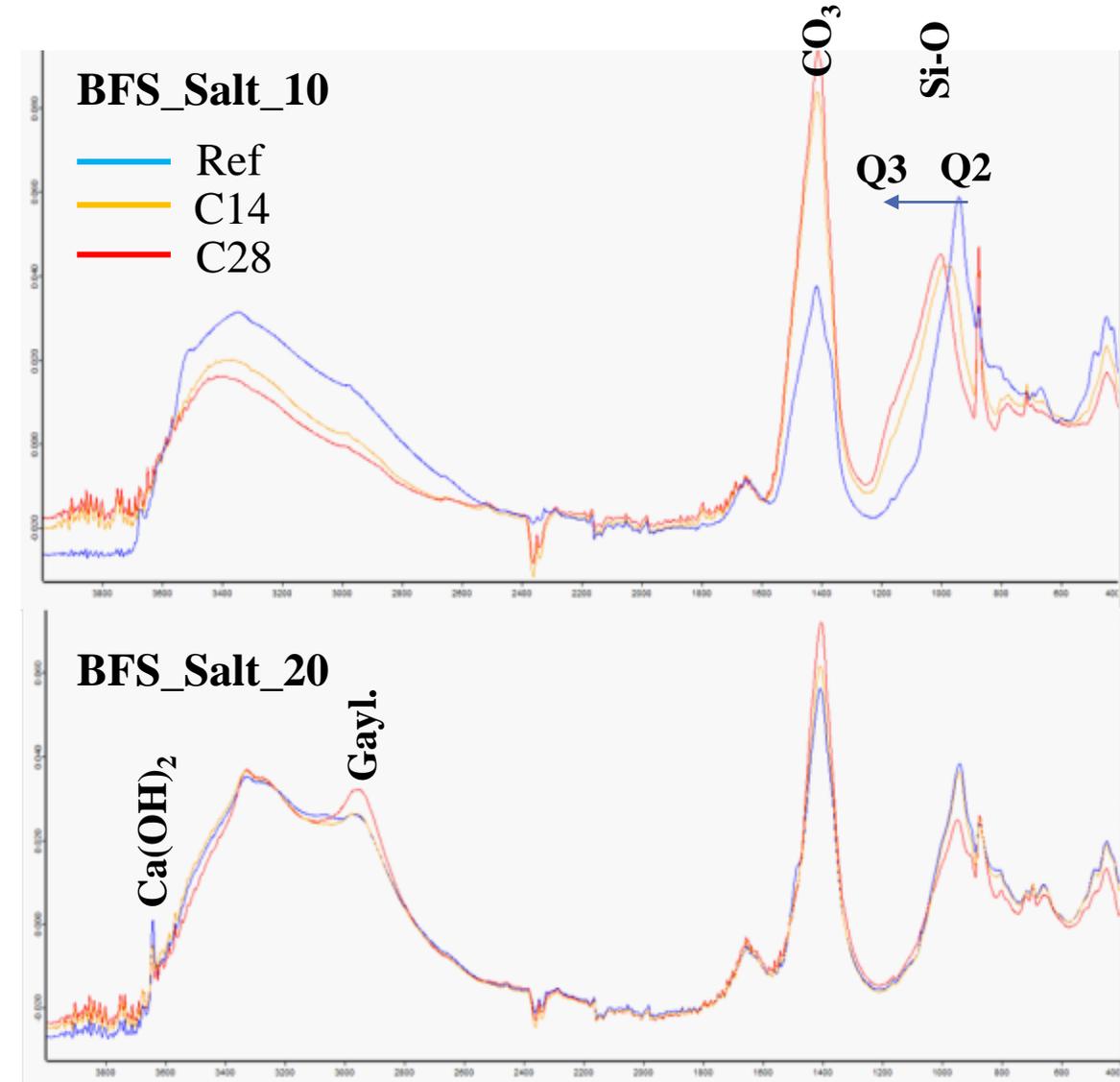
Carbonation experiments

Strength & carbonation depth



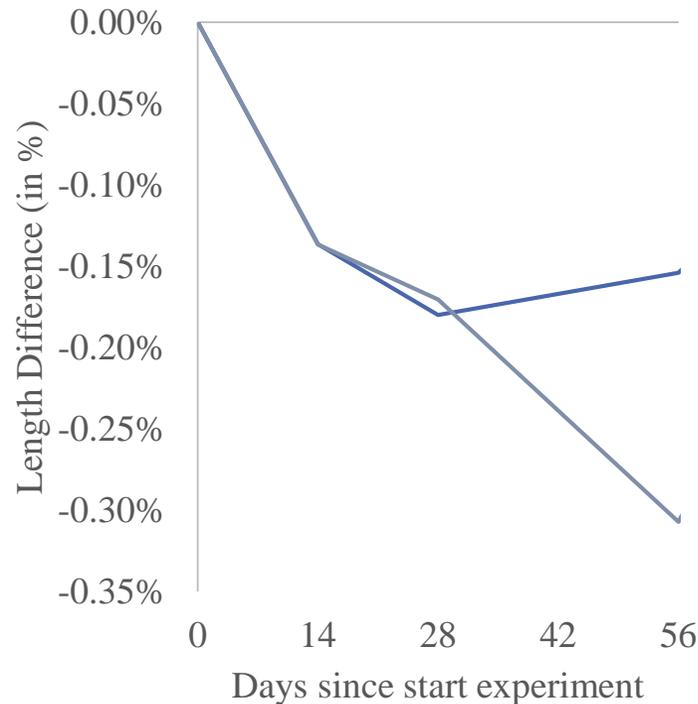
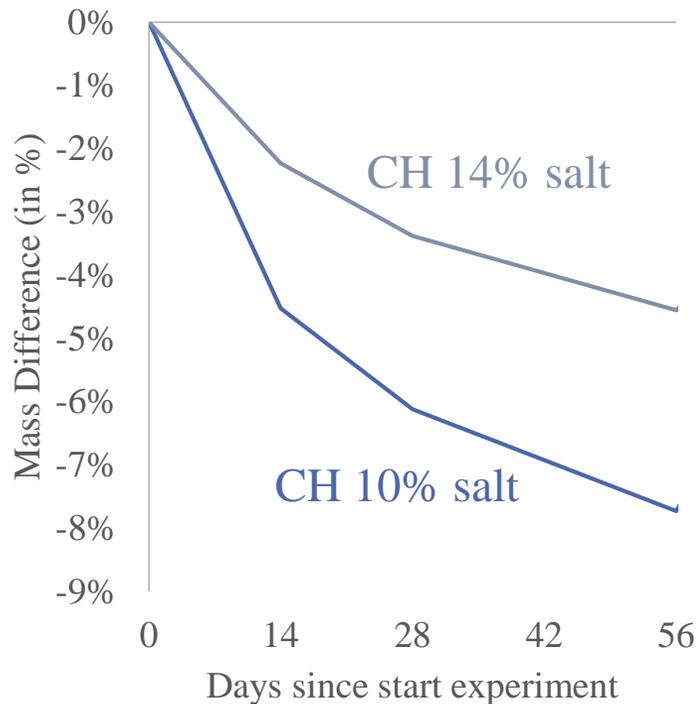


	Amorph	AFm	Cc	Gayl.	Pirss.	Ca(OH) ₂	Qz
BFS Salt 10 Ref	71	8	3	3	0	1	14
BFS Salt 10 C28	62	2	20	3	0	0	14
BFS Salt 20 Ref	52	8	0	7	13	4	15
BFS Salt 20 C28	49	8	1	14	9	3	17



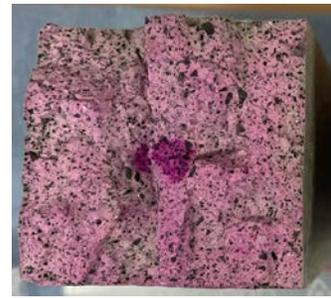
Carbonation experiments

Mass & Length



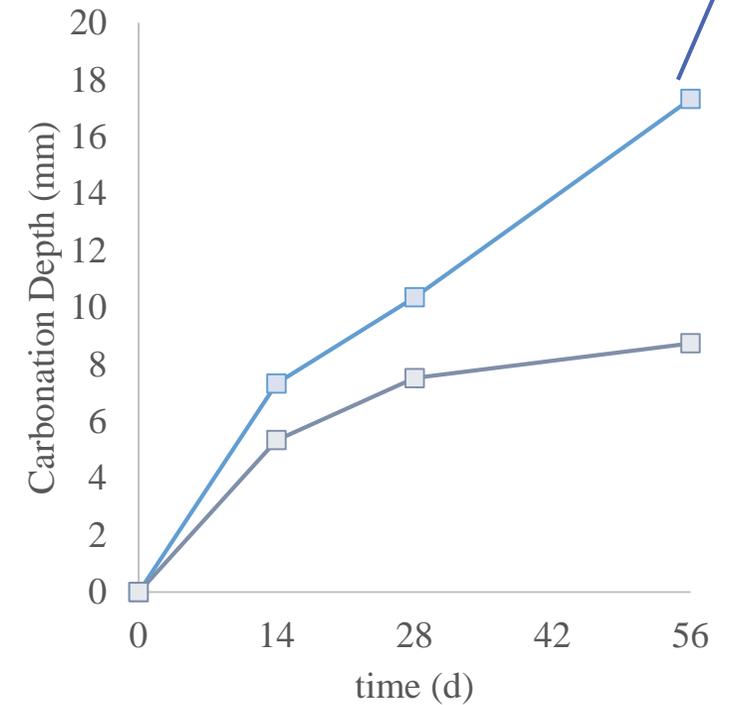
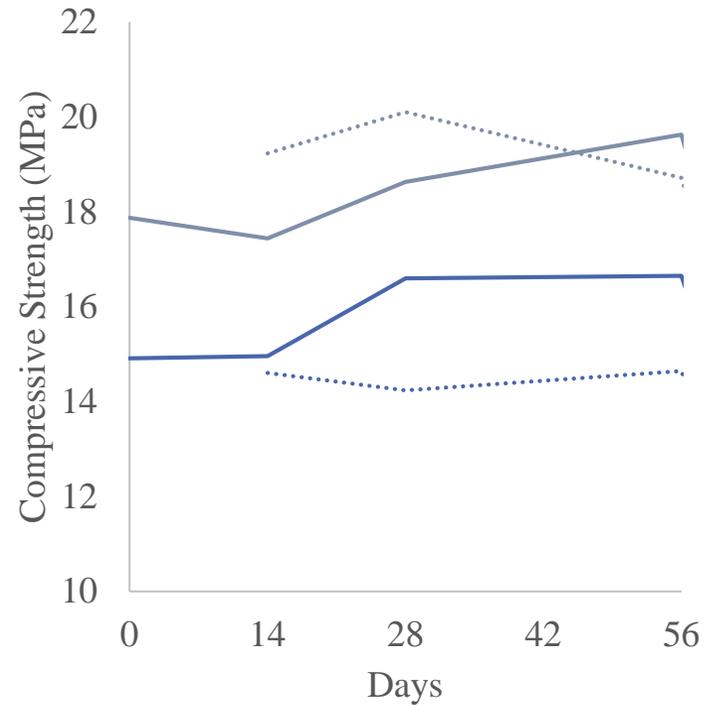
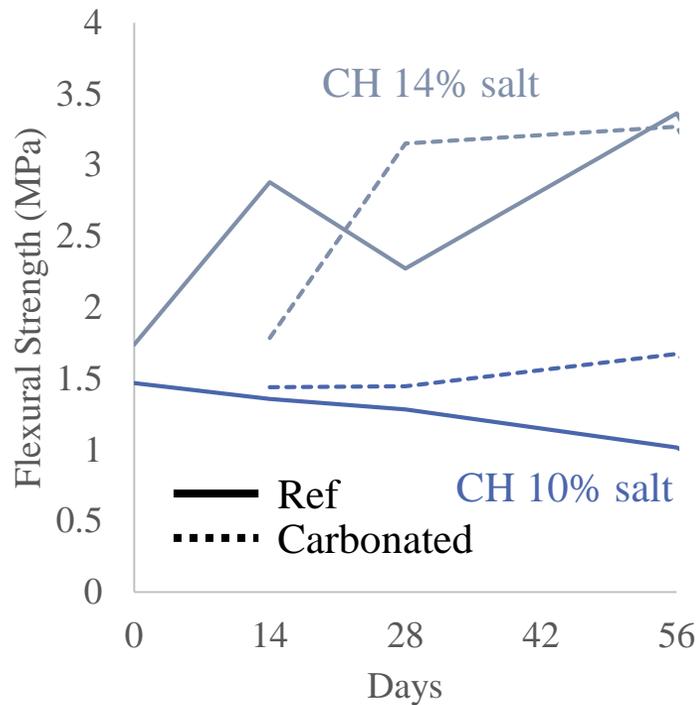
Mass decrease
 Water release > CO₂ intake
 =
 Shrinkage of samples





Carbonation experiments

Strength & carbonation depth



Leaching experiments

PREDIS 6 - protocol

- Leaching in glovebox
- Intermediary analyses leachate:
 - TIC-TOC
 - pH, resistivity
 - ICP analysis
- Leachant composition:

<i>g/l</i>	Na	Ca	K	SO4	Si	Al	pH
Expected:	0	0.02	2.80	1.09	0	0	12.70
Measured:	1.48	0.02	0.84	1.08	0.01	0	12.99

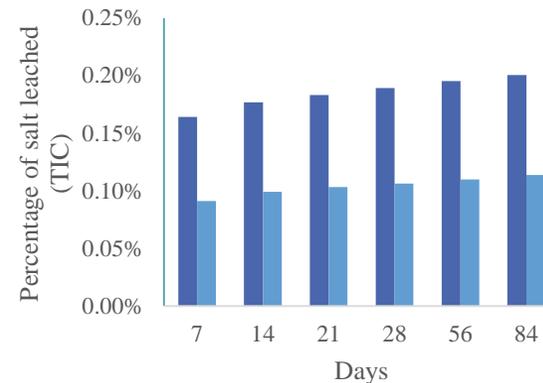
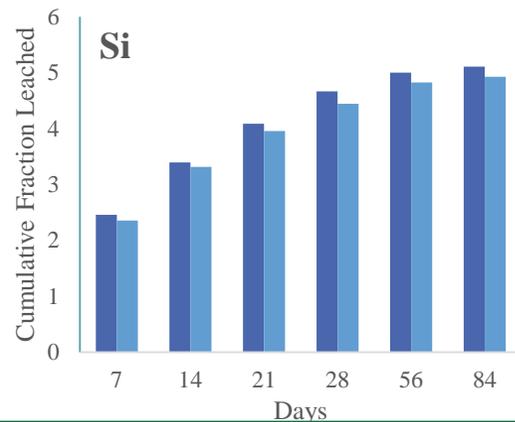
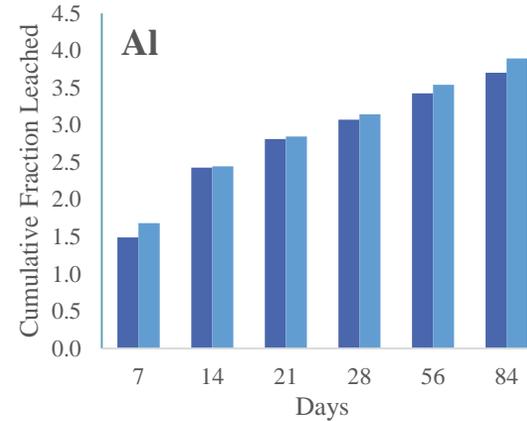
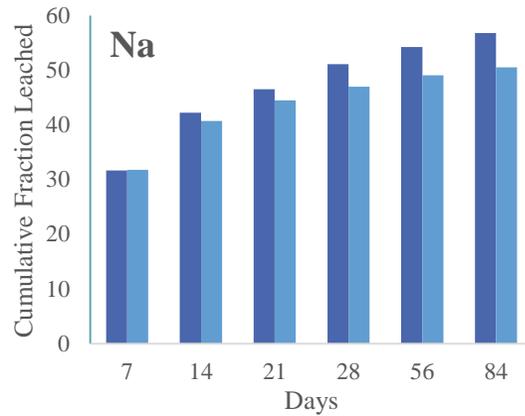
Accelerated leaching

- Leaching in reaction vessel bubbled with N₂
- Intermediary analyses leachate:
 - ICP analysis
- Leachant: 6M NH₄NO₃

Post-Mortem

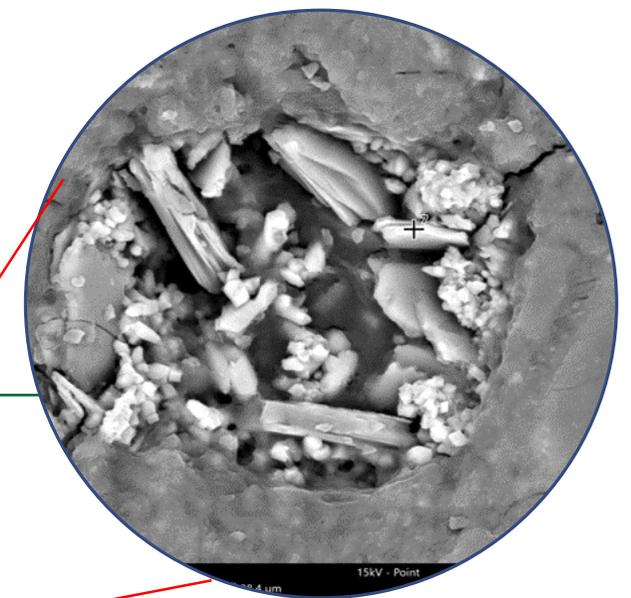
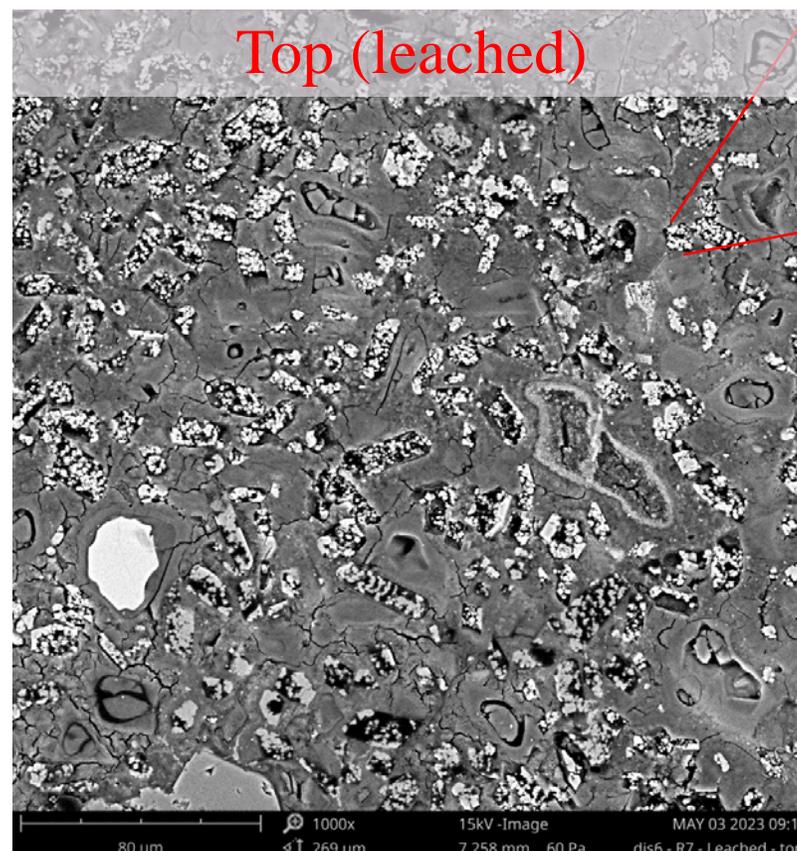
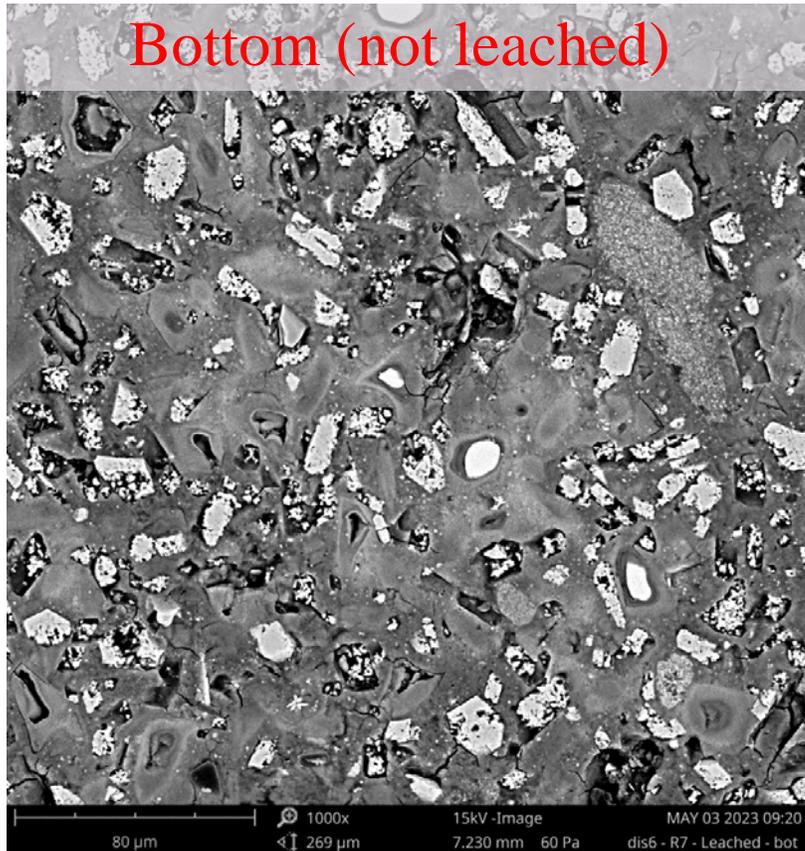
- SEM analyses
- Indentation of leached surface

Leaching – preliminary results



- Initial sample composition to be determined (estimated for CFL calculation)
- TIC and Na do not arrive at similar leaching rates

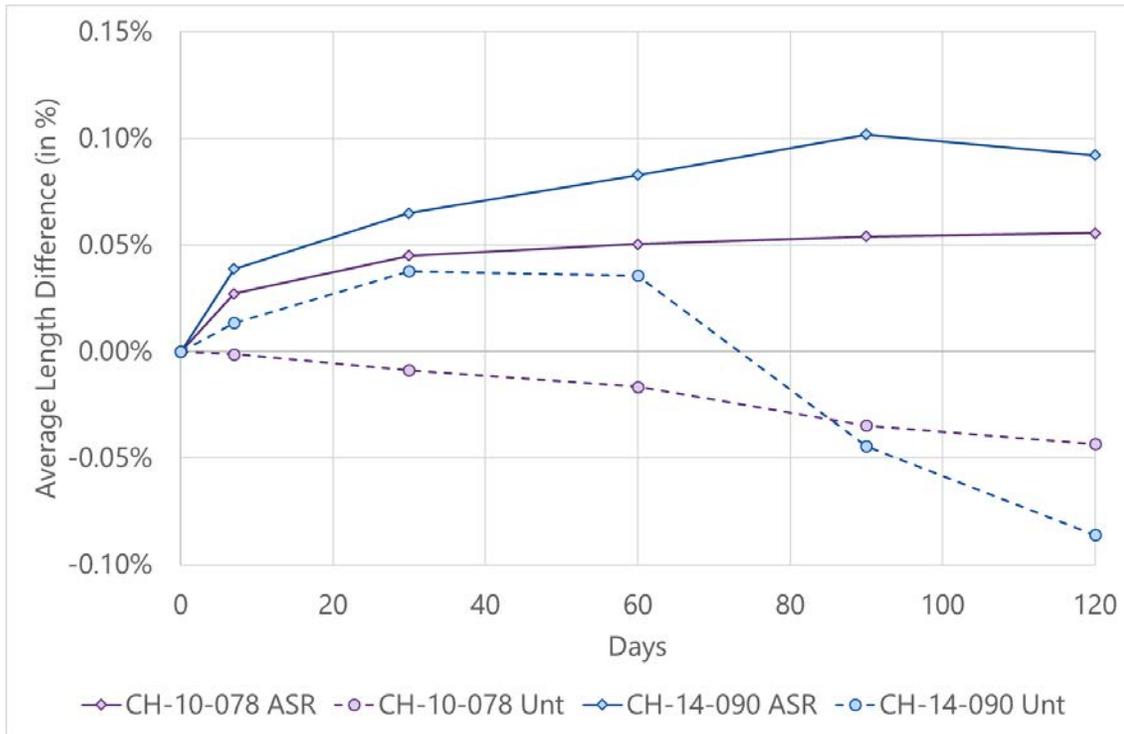
Leached samples - SEM



Carbonate crystals

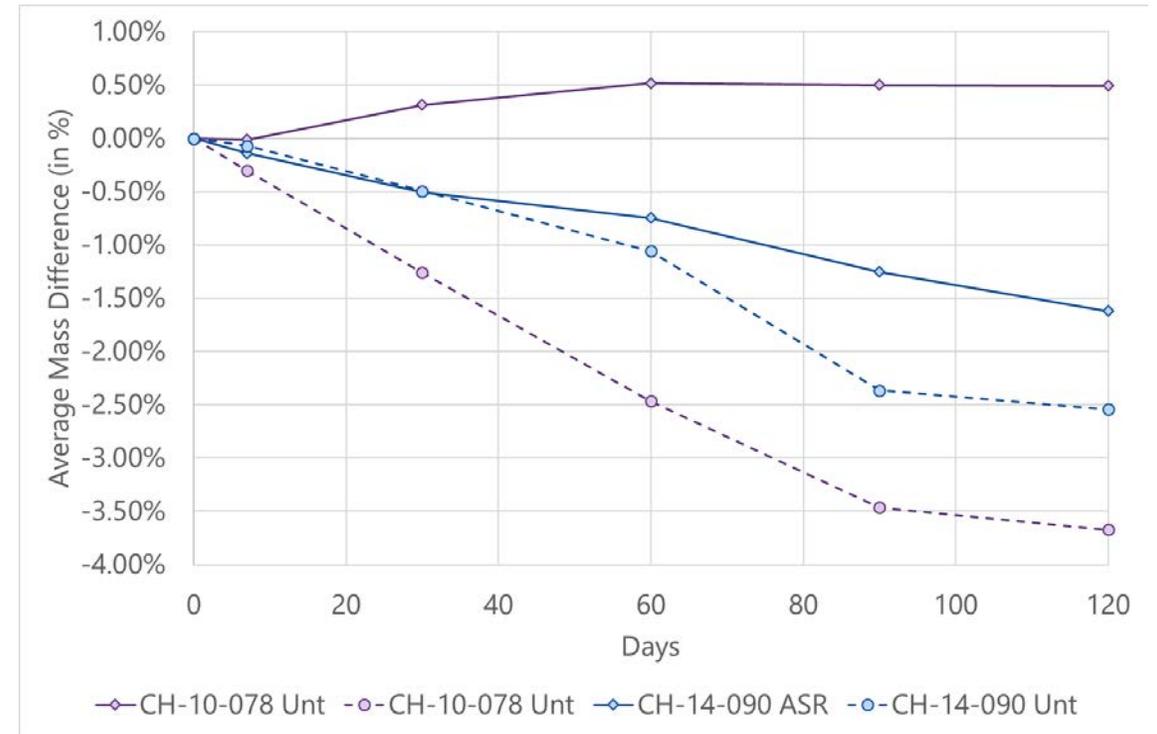
- Precipitation of carbonate crystals in voids
- Less carbonate crystals in voids near leaching front
- Degree of dissolution needs to be quantified

Long term ASR test – preliminary results



Expansion of samples under ASR

Retraction of samples in a sealed bag (Unt)



Mass loss of samples with 14 % salt

Mass gain of samples with 10 % salt



Preliminary conclusions

Alkali-activated materials

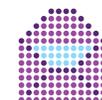
- Na-Ca carbonate (gaylussite/pirssonite) is stable in a AAM matrix
- The carbonation resistance of AAM increases with increasing waste loading due to portlandite buffering
- Alkali leaching is significant (* needs to be verified) and could be associated with dissolution of carbonate crystals in voids

Cementitious materials

- Na-Ca carbonate (gaylussite/pirssonite) is a metastable phase in cementitious materials
- The carbonation resistance also increases with increasing waste loading: further microscopic analysis after the test
- Expansion due to ASR seems limited up to 120 days.



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UNIVERSITY OF HELSINKI
FACULTY OF SCIENCE



VTT

Short-Term Stability and Physico-Chemical Characterisation of a Reconditioned Waste Form Relevant to LILW Radioactive Waste Disposal

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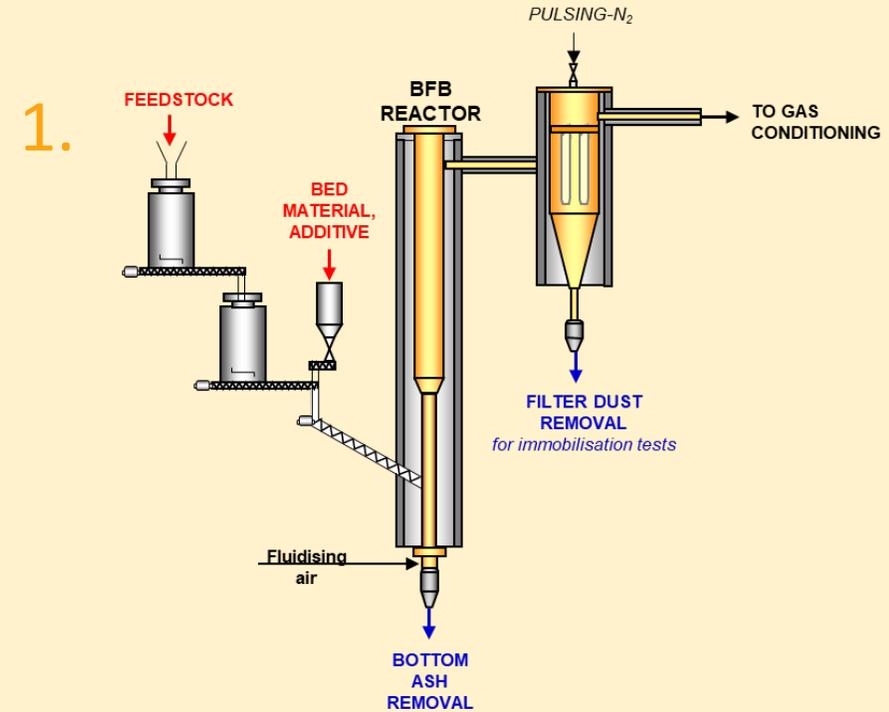


This project has received funding from the Euratom research and training programme 2019-2020 under grant agreement No 945098.

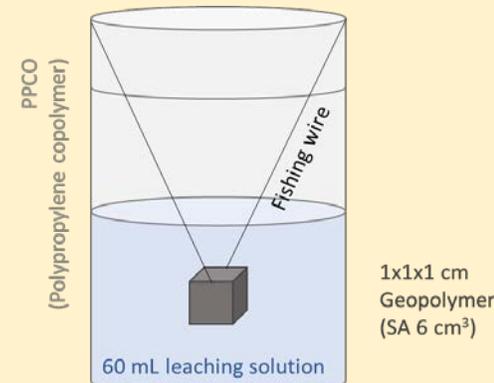


Methodology

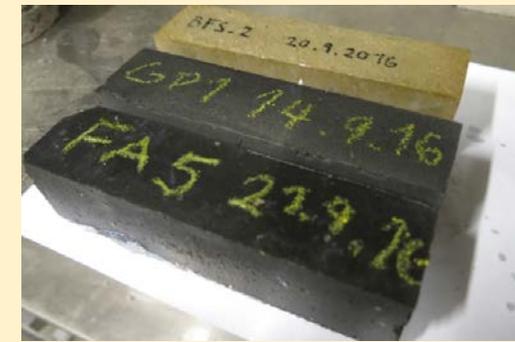
1. Gasified Ion-Exchange Resin prepared by VTT using a Bubbling fluidized bed reactor (BFB) was used for gasification of organic ion-exchange resin doped with stable RN analogues.
2. Geopolymer (1%_{wt} ash loading).
3. Leaching experiments follow PREDIS WP6 protocol.



3.



2.





Aims and Objectives

Overarching aim: Semi-dynamic leaching experiments assess the short- and long-term (1 month – 2 years) behaviour of the matrix and contaminants.

1. Pre-characterisation of the physico-chemical properties of geopolymer.

- Elemental composition.
- Structural information e.g., Imaging (SEM-EDX or EMPA) or ^{29}Si and ^{27}Al MAS NMR.

2. Assessment of leaching behavior of waste-form in repository conditions.

- Elemental composition of leachate (MPAES / ICPMS).
- Assessment of colloid formation (DLS / Zetasizer).

3. Post-mortem characterisation of the physico-chemical properties of the leached geopolymer.

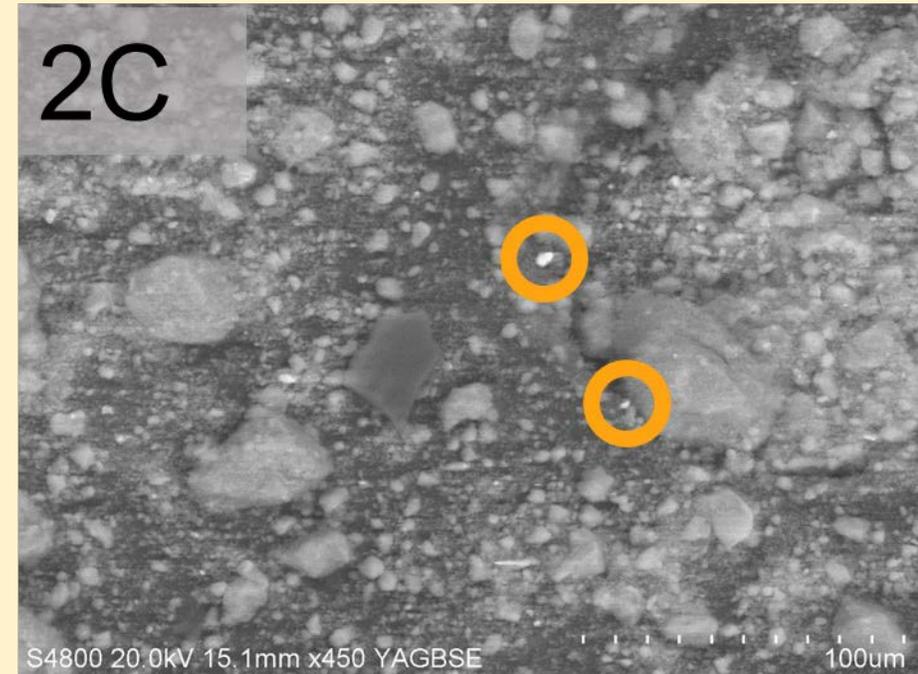
- 1, 6, 12 & 24 months.
- Elemental composition.
- Structural information e.g., Imaging (SEM-EDX or EMPA) or ^{29}Si and ^{27}Al MAS NMR.





1. Pre-characterisation

- 1% ash in a metakaolin geopolymer.
- Contains stable isotopes of: Cs (200 mg/kg), Sr (20 mg/kg) Cr (3000 mg/kg), Co (40 mg/kg), and Ni (3000 mg/kg).
- Heterogeneous particles from ash are retained after encapsulation into the geopolymer matrix.



ESEM-BSE of metakaolin geopolymers. Circles highlight dense agglomerations.





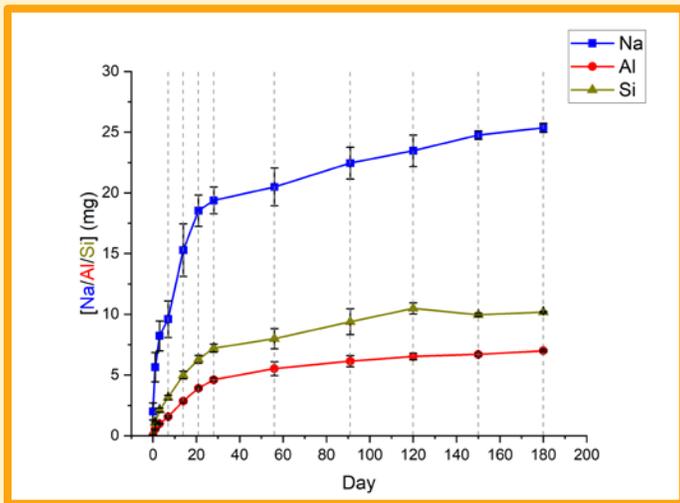
2. Aqueous Analyses

pH and **conductivity** remain **stable** at 12.5 and 8.5 mS/cm respectively. **No colloids** observed in leachate using dynamic light scattering.

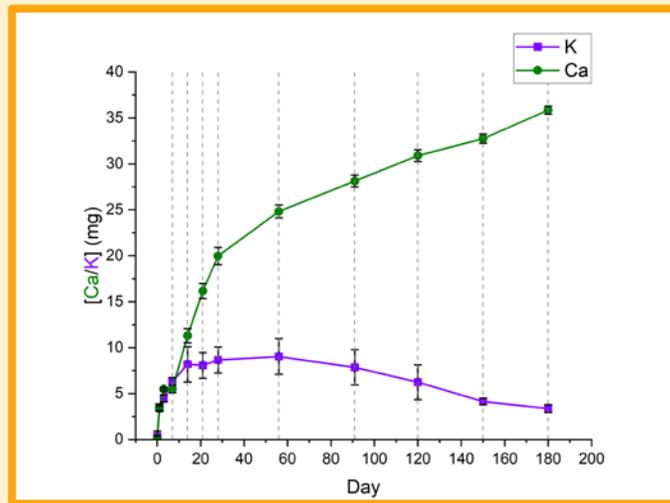
Bulk: **Na** (24.2 %), **Al** (5.9 %), and **Si** (2.9 %) leach into the solution, **Ca** (66 %) and **K** (0.8 %) are removed from solution into the geopolymer.

Trace: **Cs** (55 %) leaches into the solution. Leaching index (9.1) was significantly above waste acceptance criteria of (6.0) prescribed by US NRC for radioactive waste [ANSI 2019]. **Co**, **Cu**, **Ni**, **Sr** and **Cr** are retained within the geopolymer.

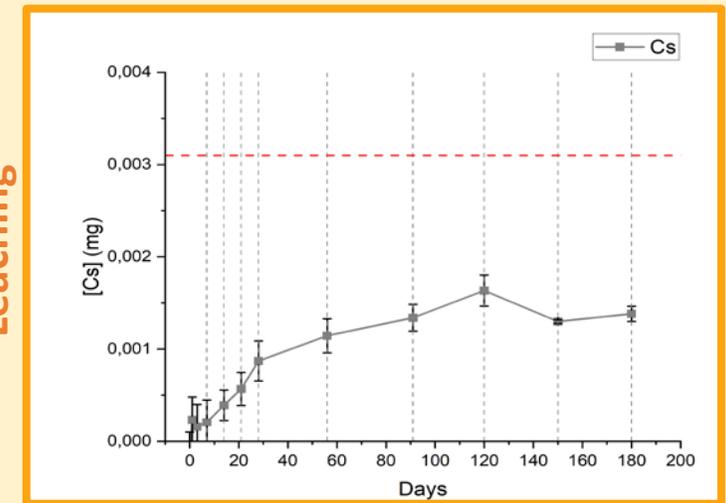
Leaching



Absorption



Leaching



[left] Cumulative **leaching** of Al, Si, and Na; [middle] Cumulative **absorption** of K and Ca; [right] Cumulative **leaching** of Cs.

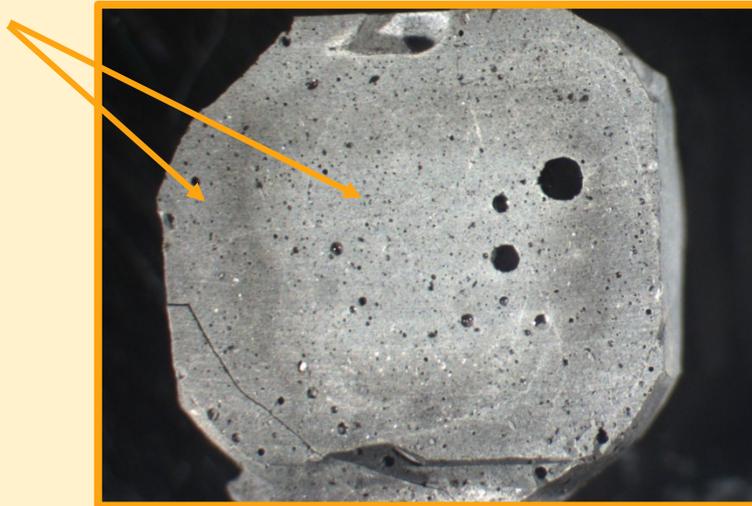




3. Post-Mortem Analyses

Leached geopolymers are sacrificed, preserved, and dissected after 1, 6, 12 and 24 months.

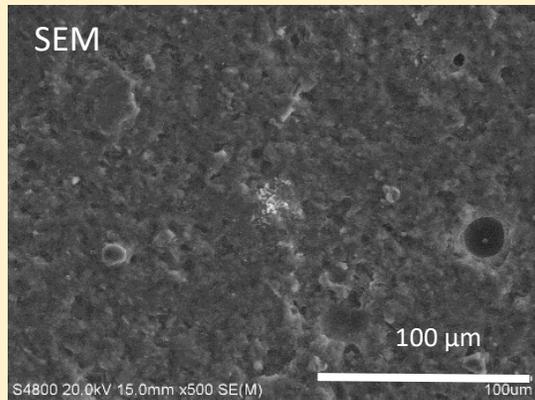
1. Different morphology
2. Weaker integrity
3. Change in geopolymer structure?





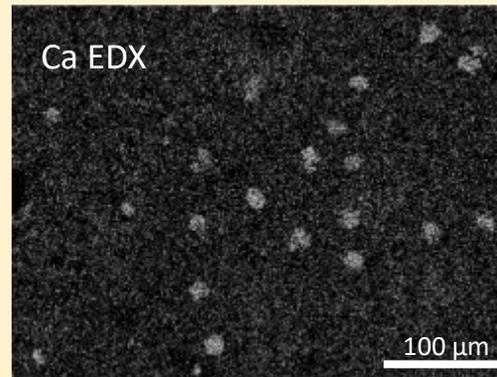
3. Post-mortem analyses

Unleached Geopolymer



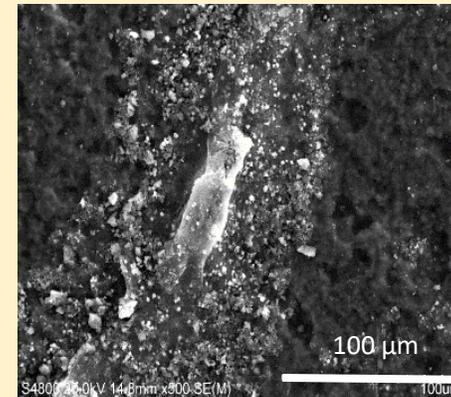
1 month

Increased cracking, change in morphology and growth of Ca rich secondary phases (5-20 μm). XRD confirmed as CaCO₃.



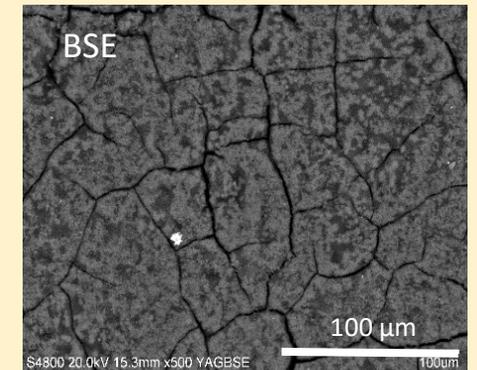
6 months

Continued change in morphology and growth of CaCO₃ secondary phases.



12 months

Continued change in morphology (now significant cracking) and growth of CaCO₃ secondary phases.





Outlook

Conclusions

- Successfully developed experimental methodology for leaching experiments + post-mortem analyses.
- Assess change in geopolymer structure and its interactions with surrogate radionuclides.
- Leaching experiments show Cs was retained within the criteria to US NRC.
- Masters thesis + Expected Publications 2023/4.

Future work

- Completion and publication of 1st gen. geopolymer experiments (12 month).
- On-going experiments with 2nd generation leaching experiments. Doped with elevated RN and ash loadings (15, 50%).
- More complex, merits molecular-scale analysis to identify bonding mechanism(s). E.g., XAS, TEM, EPMA.





PREDIS

Thank you



This project has received funding from the European Union's Horizon 2020 research and innovation programme under grant agreement No 945098.